


# A Random Walk Toward Advanced Modeling & Simulation Capabilities in Nuclear Engineering

Paul J Turinsky  
North Carolina State University



# Drivers for Nuclear Energy Advanced Modeling and Simulation Needs

- Economics
- Licensability
- Security & Safety



# Key Areas To Model

- Materials
- Fuels
- Thermal-hydraulics
- Neutronics
- Structures
- Separations & Safeguards
- Waste Form



## Common Attributes Desired of Nuclear Energy Advanced Modeling and Simulation (NEAMS)

- Multiphysics capability
  - ❖ FRR simulation requires coupling of T-H, neutronics, structures and fuels
- Multiscale capability
  - ❖ We will not be able to nor need to model every phenomena at the fine scale
- Higher-order phase space treatments
  - ❖ To achieve fidelity within computational resources
- Exploitation of leadership class computing
  - ❖ Introduction of parallel constructs and clever memory management



## Common Attributes Desired of Nuclear Energy Advanced Modeling and Simulation (NEAMS)

- Verification & Validation
  - ❖ As part of code development plan
  - ❖ Driver for experimental results required
  - ❖ Incorporating Uncertainty Quantification
- Automated Data Assimilation
  - ❖ Data adjustment (along with posterior uncertainties) based upon new experimental results
- User Friendly
  - ❖ Robustness, visualization, and automation, e.g. optimization



# Where Are We At?

## ■ Multiphysics

- ❖ Have tightly coupled core neutronics/T-H capability, with limited coupling to fuel thermo-mechanical model and other physics phenomena
- ❖ Most significant challenges are likely in fuel modeling, e.g. current codes may have eight different physics packages loosely coupled and heavily based upon correlations
- ❖ Such coupling will lead to additional complexities, e.g. strong nonlinearities, well posedness of solution, and moving material interfaces.



# Where Are We At?

## ■ Multiscale

- ❖ Space scales difficult/Time scales **super** difficult
- ❖ Have always used approach in neutronics
  - Evaluate Nuclear Data => Preprocessing (NJOY) => Resonance Treatment => Lattice Physics => Core Simulator
  - Approach: Start with great energy, angular and spatial detail for small spatial subdomains and end with little energy, angular and spatial detail for the large spatial domain.
  - **Trouble: One-way street problem!**
  - ✓ Some interesting ideas exist using subspace methods to formulate a two-way street approach while achieving consistent closure.
  - ✓ The curse of resonances, which are approximately treated.
- ❖ Evolving capabilities in thermal-hydraulics (DNS to Components) and materials (ab initio to fracture mechanics).



# Where Are We At?

## ■ Higher-order Phase Space Treatments

- ❖ Temporal: Abandoning operator splitting approaches for multiphysics problems to utilize higher-order treatments
- ❖ Energy: Expert judgement on energy group structure still prevails. There has to be a better approach!
- Now can calculate scattering kernels using ab initio code to get classical potential function followed by use in molecular dynamics code. But we still cannot calculate cross-sections without considerable approximation, e.g. optical models, except for simplest nuclei.
- ❖ Spatial: Lots of alternatives with higher-order methods.
- Many times mesh refinement is preferred approach due to material heterogeneities. However, continued mesh refinement may invalidate physics captured in model being used, e.g. diffusion theory and fluid flow.
- ❖ Angular (neutronics): Several alternatives developed to address, e.g. quasi-diffusion theory & generalized equivalence theory.





# Where Are We At?

- Exploitation of leadership class computing
  - ❖ Nuclear energy enterprise used to lead in using latest high-end computing resources, but now other enterprises lead, e.g. aeronautics, pharmaceutical, automotive, and weather forecasting.
  - ❖ Lots to learn from other activities, e.g. ASC & SciDAC
  - ❖ Needed at least in research phase, many times in support of developing understanding & coarser-scale models, and supporting V&V.

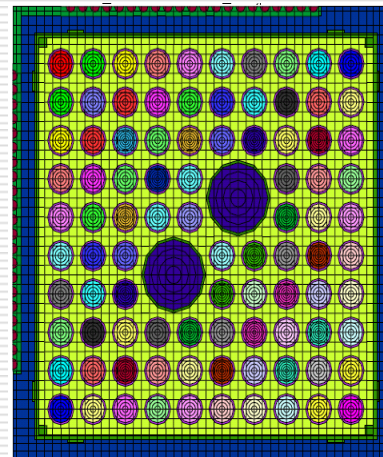
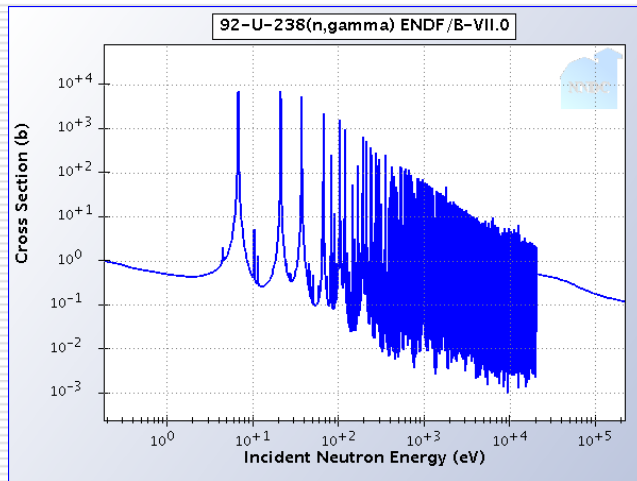


# Where Are We At?

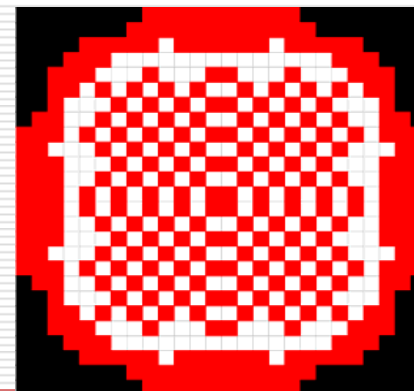
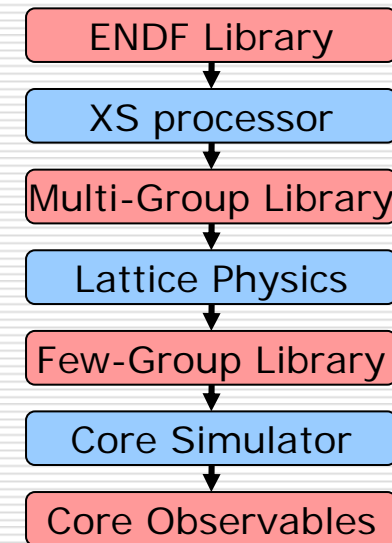
## ■ Verification & Validation

- ❖ Without industry and regulatory acceptance is doomed.
- ❖ Error & Uncertainty Quantification: Desire knowledge of errors & uncertainties based upon source, i.e. numerical treatment, modeling, epistemic uncertainties (e.g. data including correlations), aleatory uncertainties (random phenomena), and initial & boundary conditions.
- ✓ Until recently could address data uncertainties only if data field small (forward perturbation [DAKOTA] or Perturbed PDEs [SUNDIAL]) or response field small (adjoint based method for linear problems). Now via Efficient Subspace Method (ESM) can address simultaneous large data and response fields.
- ✓ Data uncertainties (including correlations) dependence on state condition needs further attention.

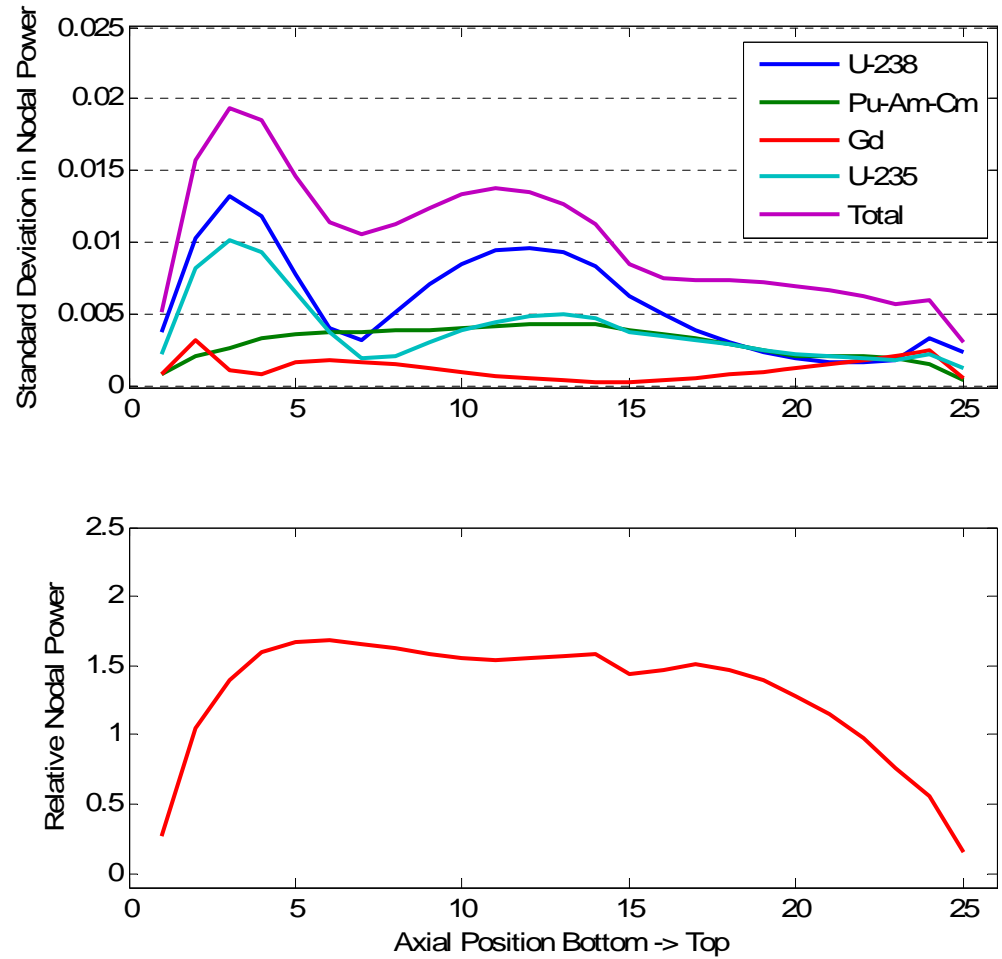
# BWR Calculational Sequence



GE14 10x10 lattice design



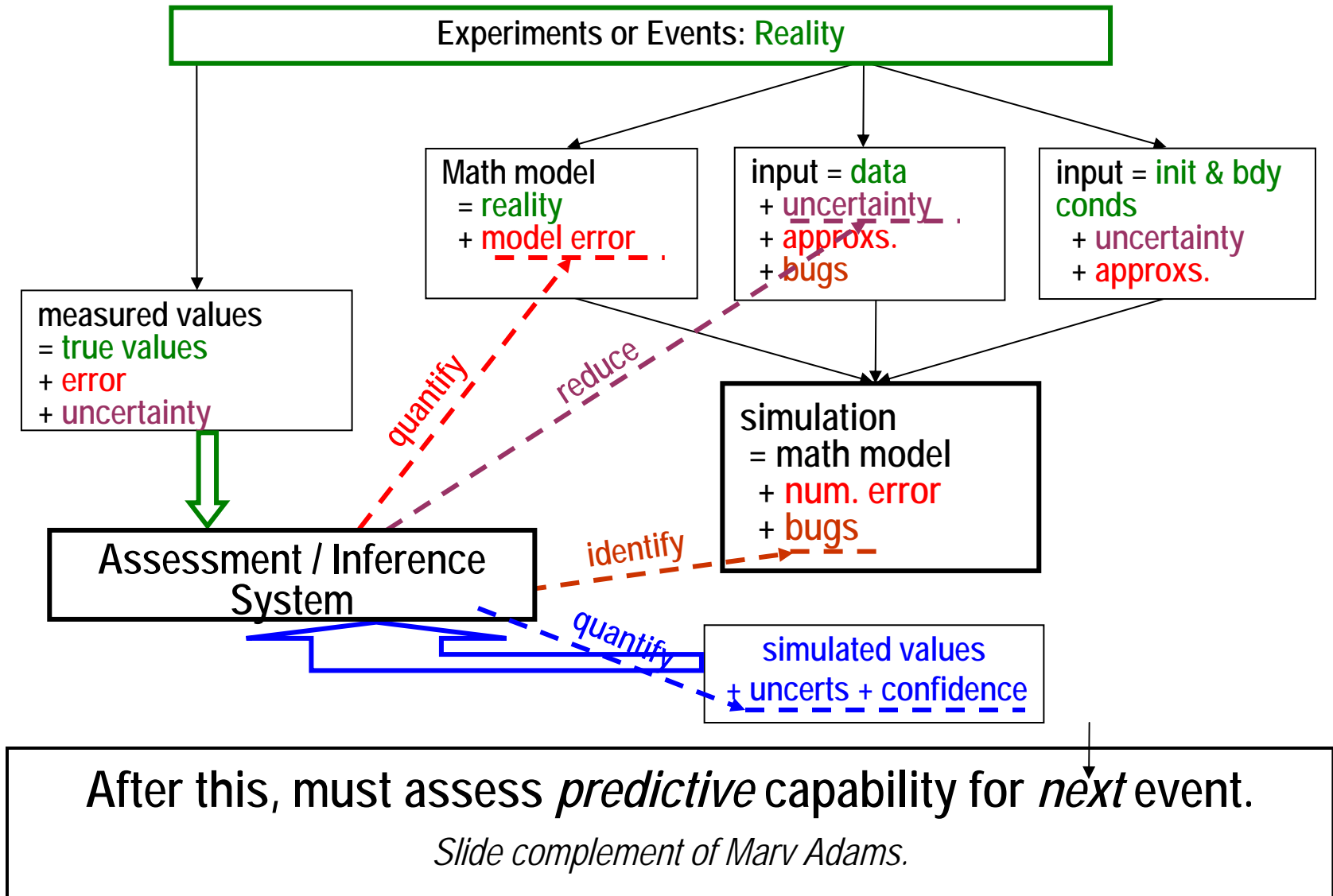
# FG Cross-section UP



BOL Uncertainty Profile



# We must go beyond traditional V&V and UQ.





# Where Are We At?

## ■ Verification & Validation

- ❖ Coarser scale models and numerical errors can be addressed via adjoint based method to 1<sup>st</sup> order accuracy, but not commonly done.
- ❖ Little work done on B.C. & I.C. introduced uncertainties, though amendable to adjoint based method when linear responses.

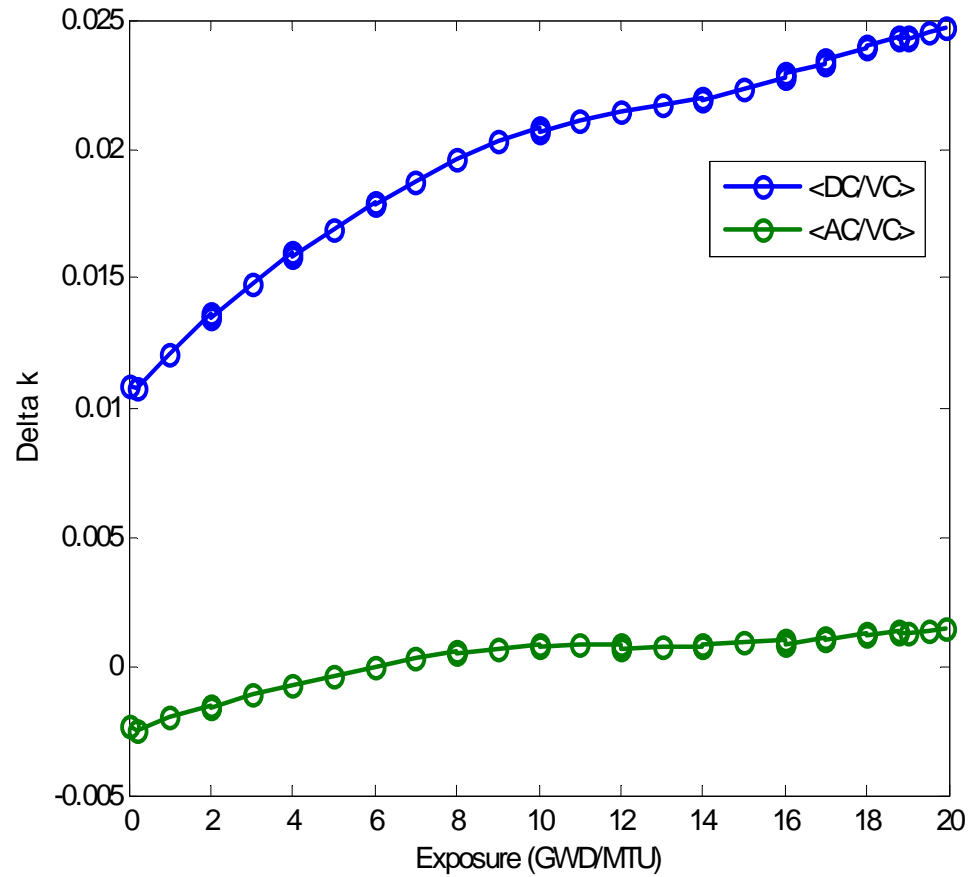


# Where Are We At?

## ■ Automated Data Assimilation

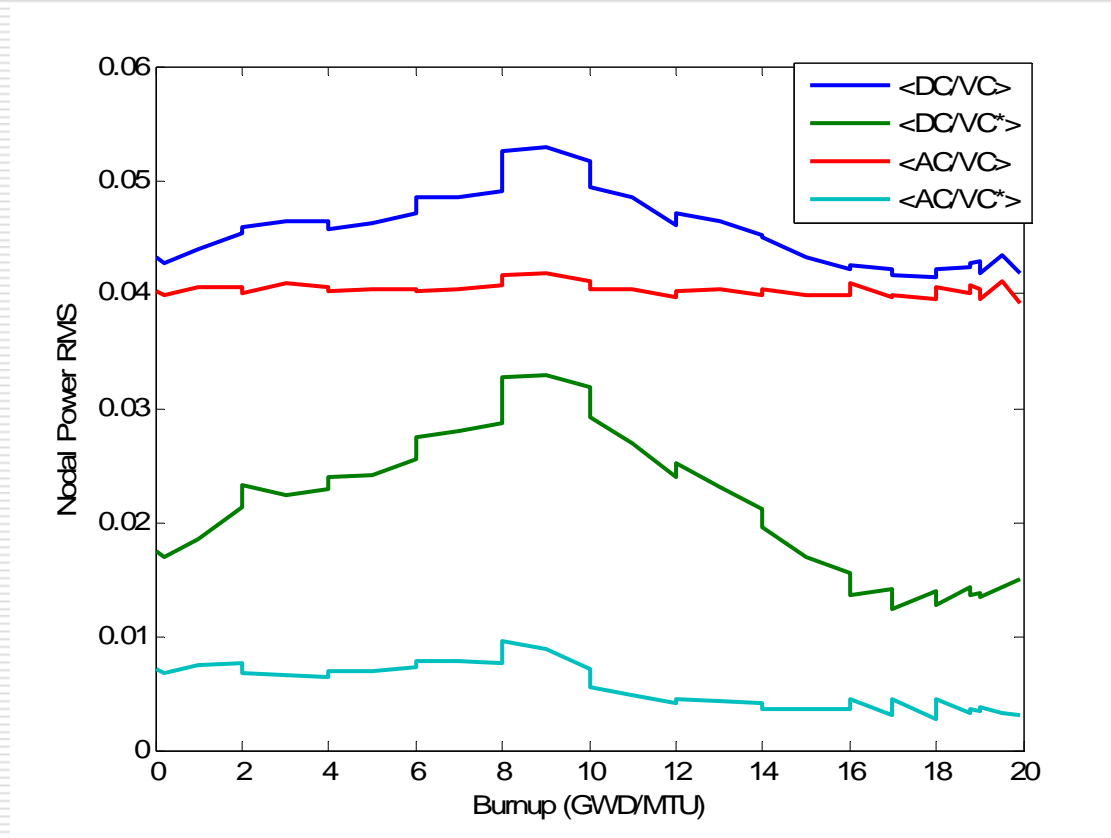
- ❖ Needs to be part of code development effort, not an after thought.
- ❖ Nuclear enterprise lags far behind other application areas, e.g. weather forecasting, so no systematic mining of data sources, e.g. plant data.
- ❖ Capabilities well known for linear observables with normal distributions
- ❖ For nonlinear observables and non-normal distributions, some capabilities exist but less well known and hence less utilized by nuclear energy related codes.

# Adaptive Core Simulation

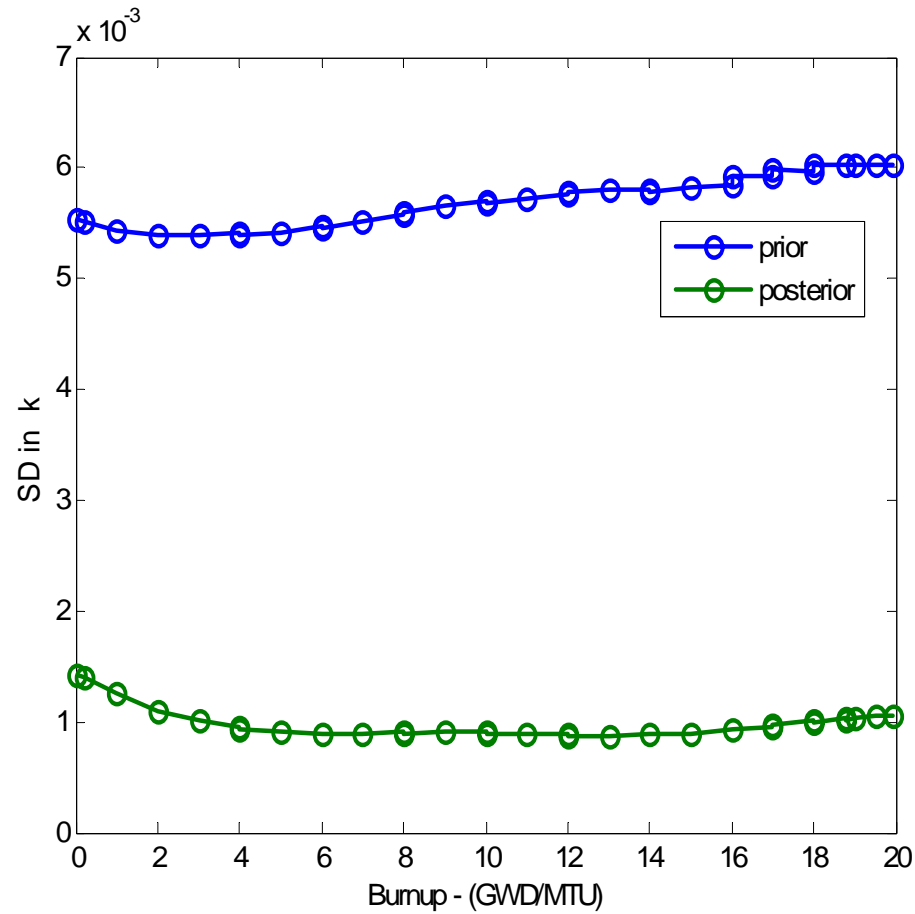




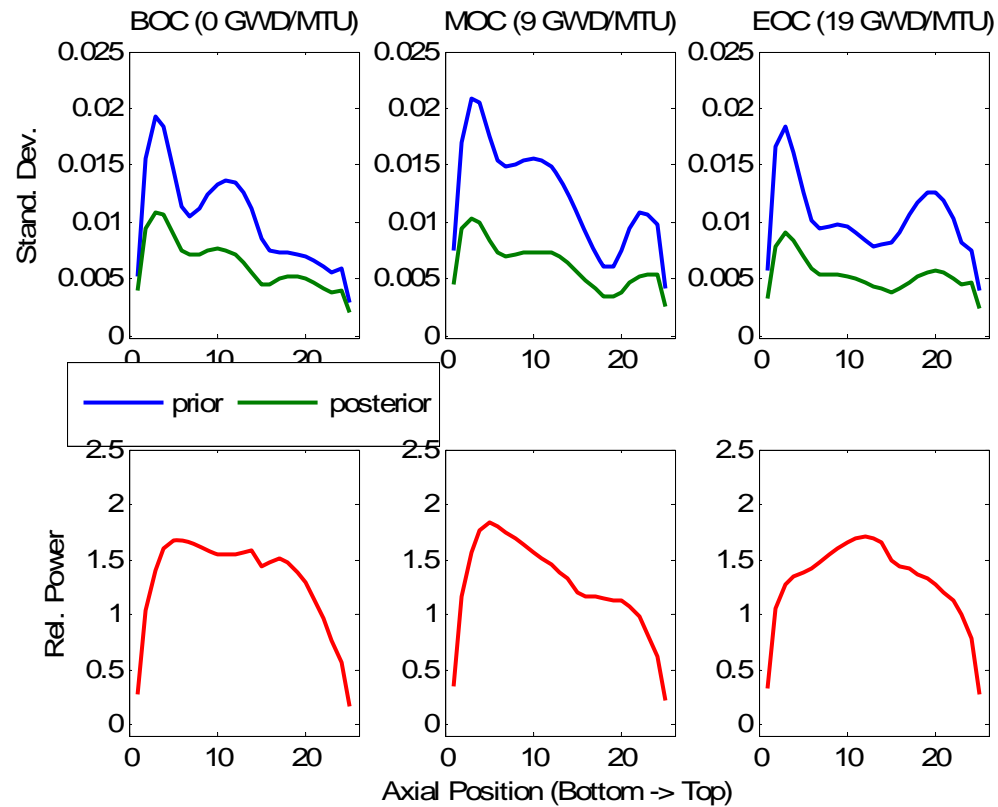
# Adaptive Core Simulation



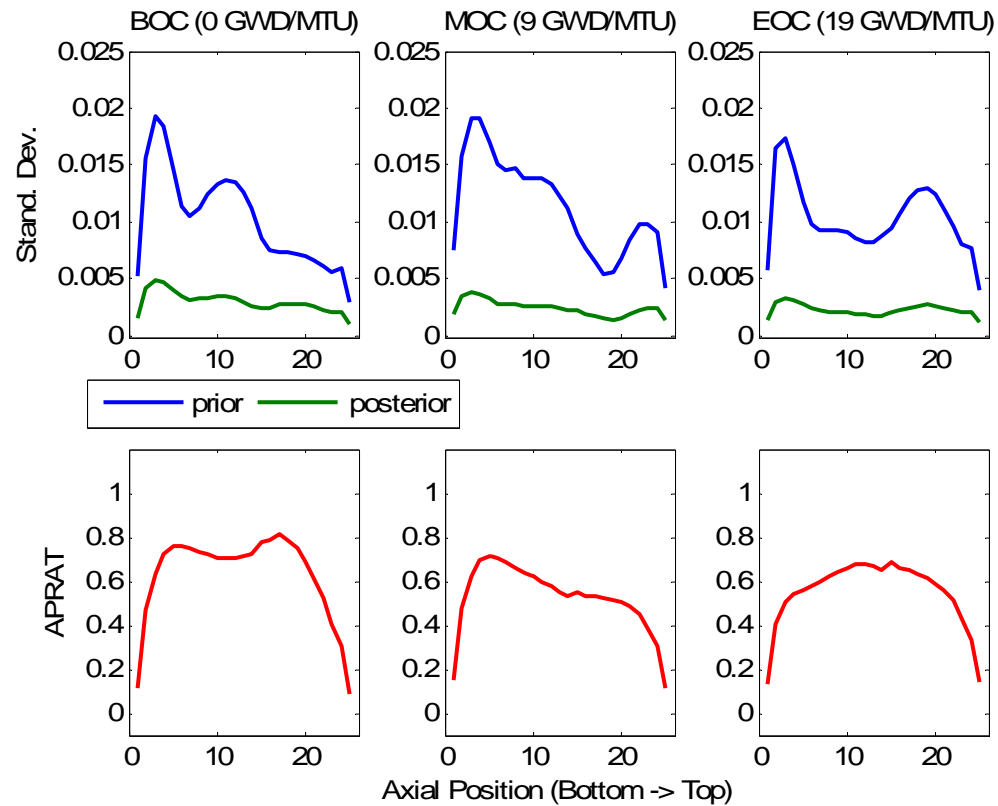
# Adaptive Core Simulation



# Adaptive Core Simulation



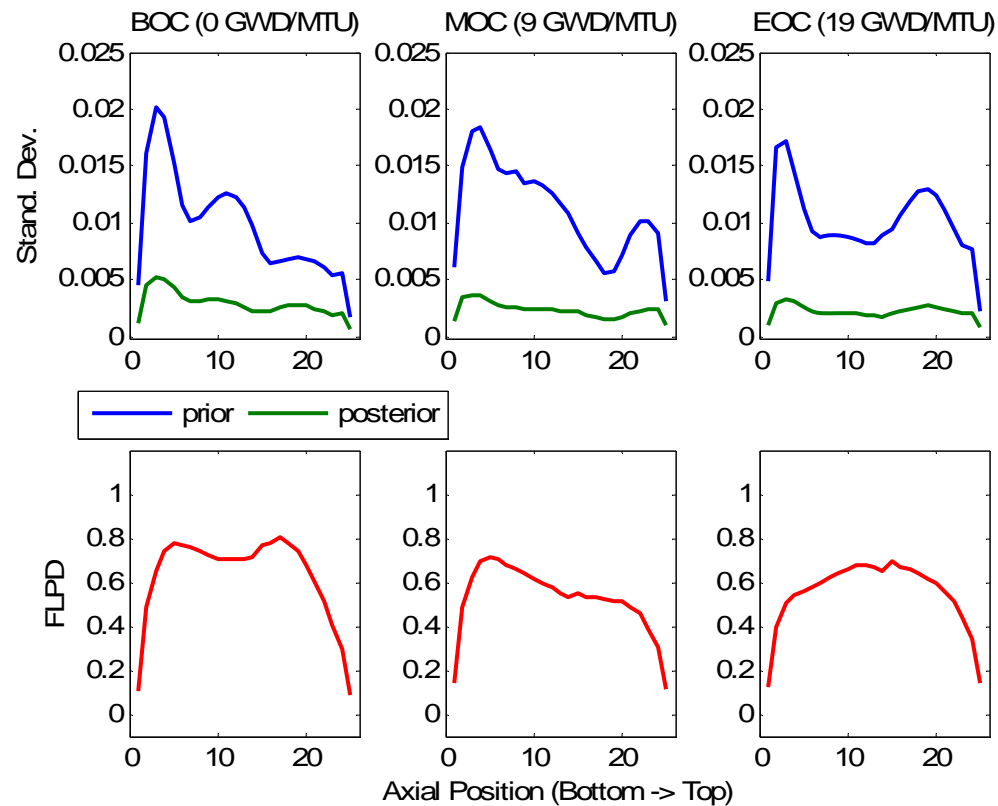
# Adaptive Core Simulation



*APRAT: Average Linear Power Density Ratio*



# Adaptive Core Simulation



*FLPD: Fraction to Limiting Power Density*





# Where Are We At?

## ■ User Friendly

- ❖ Visualization is being addressed by commercial software in discipline specific framework.
- ❖ Robustness via numerical algorithms is being addressed in V&V activities.
- ❖ Automation: Optimization of complex phenomena will require mathematical optimization. Already being used routinely in incore fuel management optimization ( $10^{50}$  decision space) and fuel cycle optimization, but needs extension to other areas of application.



# Where Are We Going Now?

- By DOE laboratory structure => Diverse nuclear energy program.

## EXAMPLES

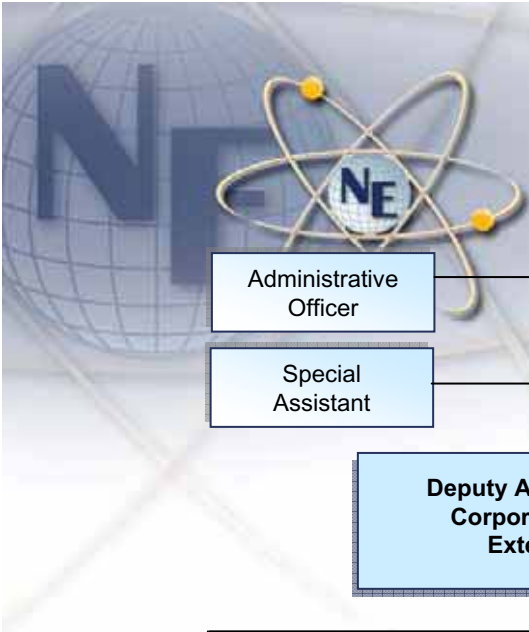
- ❖ ANL: Focus on core analysis via multiphysics code, i.e. Numerical Simulator.  
Have developed advanced neutronics capabilities, i.e. UNIC (Ultimate Neutronic Investigation Code)
- ❖ INL: Learning experiences on fuel and core multiphysics/multiscale with AMoR and UQ.  
RELAP7 development about to be implemented.
- ❖ ORNL: Extending SUNAMI capabilities beyond criticality.



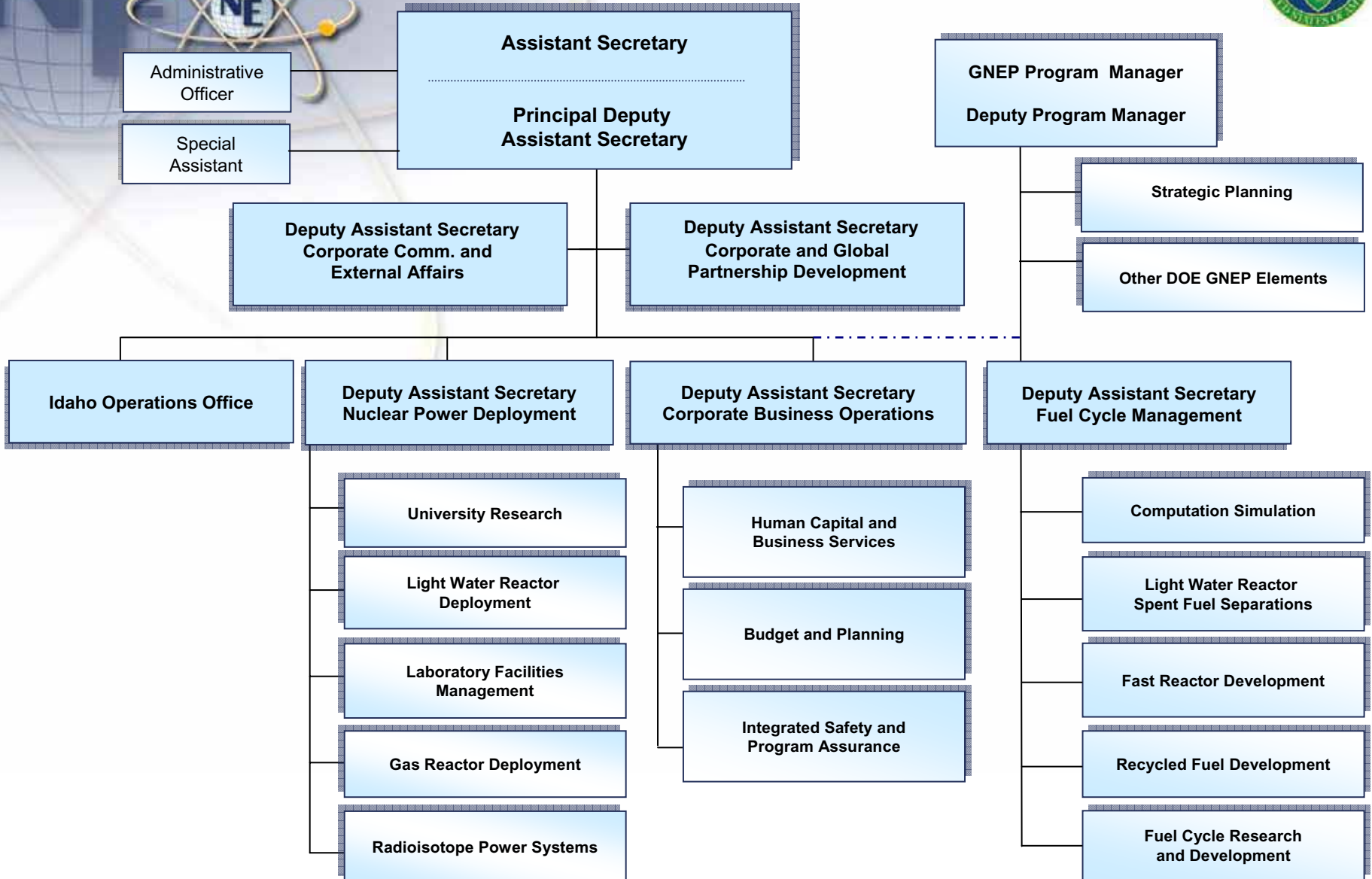
# Where Are We Going In The Future?

- Nuclear Energy Advanced Modeling and Simulation (NEAMS) cross-cut activity within the GNEP program.

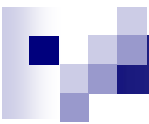




# Office of Nuclear Energy



— Operational Chain of Command    - - - - Administrative Chain of Command



# Nuclear Energy Advanced Modeling & Simulation NEAMS

## ■ *Vision*

To rapidly create, and deploy “science-based” verified and validated modeling and simulation capabilities essential for the design, implementation, and operation of future nuclear energy systems with the goal of improving future U.S. energy security.

## ■ *Approach*

- Produce the new modeling and simulation capabilities with appropriate flexibility to allow them to be applicable to a variety of nuclear energy system options and fuel cycles
- Continuously deliver improved modeling and simulation capabilities relevant to existing and future nuclear systems (in the near, mid, and long term)
- Apply the best ideas through open, competitive processes to the challenges of achieving the NEAMS vision



# NEAMS Foundation

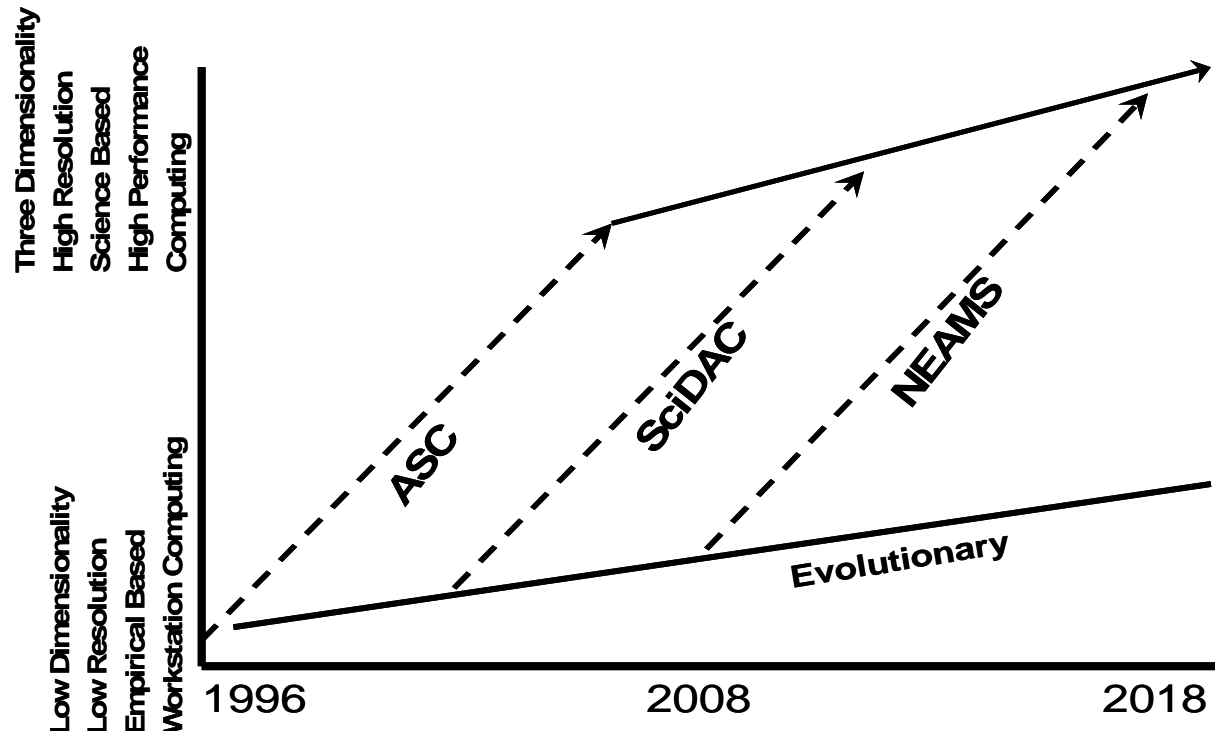
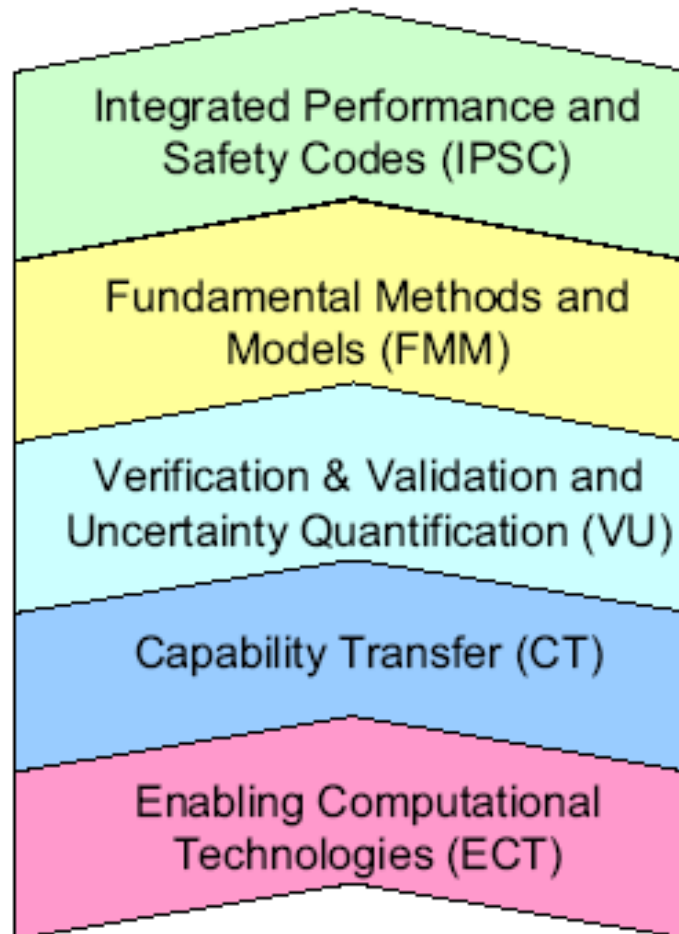


Figure 4: History of Leaping to New Levels of Modeling and Simulation Capability

# NEAMS Program Elements





# NEAMS Program Elements

## ❖ **Integrated Performance and Safety Codes (IPSC)**

- Reactor Performance and Safety Simulations that include:
  - Transmutation Fuels Performance
  - Nuclear core performance
  - Balance of plant operations and safety
- Separations and Safeguards
- Waste Forms and Repositories



# NEAMS Program Elements

- ❖ **Fundamental Methods and Models (FMM)**

Smaller length scale material modeling work, and Atomistic-to-Continuum (AtC) multi-scale simulation supported by experiments



# NEAMS Program Elements

- ❖ **Verification & Validation and Uncertainty Quantification (VU)**
  - Provide confidence that results are a prediction of the “real world.”
  - Develop & implement methodologies to understand margins & uncertainties associated with simulation results.



# NEAMS Program Elements

## ❖ **Capability Transition (CT)**

Provide necessary pathways to get capabilities out of R&D world & into hands of the end users.





# NEAMS Program Elements

## ❖ **Enabling Computational Technologies (ECT)**

Ensure that enabling technologies are available to make the first four program elements possible.

- advanced algorithms and solvers
- programming debuggers
- code performance analyzers
- model setup
- results analysis (e.g. visualization)

Also includes platforms that will be required to support the code development and the application work.

<b>Program Element</b>	<b>FY-09</b>	<b>FY-10</b>	<b>FY-11</b>	<b>FY-12</b>	<b>FY-13</b>
Integrated Performance and Safety Codes (IPSC)	\$40M	\$70M	\$90M	\$100M	\$120M
Fundamental Methods & Models (FMM)	\$8M	\$10M	\$15M	\$20M	\$25M
V&V and UQ (VU)	\$5M	\$10M	\$15M	\$25M	\$30M
Capability Transfer (CT)	\$1M	\$5M	\$10M	\$12M	\$15M
Enabling Computational Technologies (ECT)	\$1M	\$5M	\$40M	\$50M	\$60M
<b>Totals</b>	<b>\$55M</b>	<b>\$100M</b>	<b>\$170M</b>	<b>\$207M</b>	<b>\$250M</b>



# LWR Sustainability Program

## ***Program Elements***

- 1. Nuclear Materials Aging and Degradation.***
- 2. Advanced LWR Fuel Development.***
- 3. Risk-Informed Safety Margin  
Characterization.***
- 4. Advanced Instrumentation and Control  
Technologies***