

Burnup credit bibliography

Publicly available documents

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A selection of documents assembled until the end of 2005 for the Swedish Nuclear Power Inspectorate under contract 2005/705-20050619

This list is intended as a contribution to the OECD/NEA NSC WPNCS Expert Group on Burnup Credit

Even though the documents are publicly available they are often copyrighted and require some payment. Many documents are available electronically on the internet but no effort has been made to include the sources for such documents.

Many documents that are not publicly available have been excluded. In particular, licensing support documents (even though some are publicly available, particularly in the U.S.A.) and working documents from various meetings and studies are excluded. Presentation material (Microsoft PowerPoint etc.) from various meetings may be publicly available but are not included.

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Litteratur

The list has been structured to simplify search.

OECD/NEA

Final reports from Burnup Credit Expert Group

1. NEA/NSC/DOC(93)/22 (JAERI-M 94-003), M. Takano, *OECD/NEA Burnup Credit Criticality Benchmark – Result of Phase IA*, January 1994.
2. NEA/NSC/DOC(96)/06 (ORNL/TM-6901), M. D. DeHart, M. C. Brady, C. V. Parks, *OECD/NEA Burnup Credit Criticality Benchmark – Phase IB Results*, June 1996.
3. NEA/NSC/DOC(96)/01 (JAERI-Research 96-003), M. Takano, H. Okuno, *OECD/NEA Burnup Credit Criticality Benchmark – Result of Phase IIA*, February 1996.
4. NEA/NSC/DOC(1998)1 (IPSN/98-05), A. Nouri, *Burnup Credit Criticality Benchmark: Analysis of Phase II-B Results of a Conceptual PWR Spent Fuel Transportation Cask*, May 1998.
5. NEA/NSC/DOC(2000)12, (JAERI-Research 2000-0441), H. Okuno, Y. Naito, Y. Ando, *OECD/NEA Burn-up Credit Criticality Benchmark Phase IIIA: Criticality Calculations of BWR Spent Fuel Assemblies in Storage and Transport*, September 2000.
6. NEA/NSC/DOC(2002)2 (JAERI-Research 2002-001), H. Okuno, Y. Naito, K. Suyama, *OECD/NEA Burn-up Credit Criticality Benchmarks Phase IIIB: Burnup Calculations of BWR Spent Fuel Assemblies for Storage and Transport*, February 2002.
7. NEA/NSC/DOC(2003)3, G. J. O'Connor, R. L. Bowden, P. R. Thorne, *Burn-up Credit Criticality Benchmark, Phase IV-A: Reactivity Prediction Calculations for Infinite Arrays of PWR MOX Fuel Pin Cells*, 2003.
8. NEA/NSC/DOC(2003)4, G. J. O'Connor, P. H. Liem, *Burn-up Credit Criticality Benchmark, Phase IV-B: Results and Analysis of MOX Fuel Depletion Calculations*, 2003.

ICSBEP Handbook

Only a few benchmarks are directly referring to burnup credit but many others are important to burnup credit application.

9. ICSBEP Handbook/2005, NEA/NSC/DOC(95)03, International Criticality Safety Benchmark Evaluation Project, *International Handbook of Evaluated Criticality Safety Benchmark Experiments*, September 2005. Updated yearly..
10. ICSBEP Handbook/2005/LEU-COMP-THERM-050, Vol IV, J. A. Anno, ¹⁴⁹Sm *Solution in the Middle of Water*, September 2000 (latest update).
11. ICSBEP Handbook/2005/LEU-COMP-THERM-079, Vol IV, G. A. Harms, *Water-Moderated U(4.31)O₂ Fuel Rod Lattices Containing Rhodium Foils*, September 2005 (latest update).

IAEA – Technical Committee Meetings on burnup credit

The reports from the Consultancy Meetings between the Technical Meetings have been excluded from the list.

1997 Proceedings of Advisory Group – Vienna, 20-24 October

12. IAEA-TECDOC-1013/1, *Summary - Implementation of Burn-Up Credit Spent Fuel Management Systems*, October 1997.
13. IAEA-TECDOC-1013/2, M. Manolova, *Burnup credit implementation in WWER spent fuel management systems: Status and future aspects*, October 1997.
14. IAEA-TECDOC-1013/3, L. Markova, *Status of the development of burnup credit in the Czech Republic*, October 1997.
15. IAEA-TECDOC-1013/4, Y. Chanzy, E. Guillou, *COGEMA/TRANSNUCLEAIRE's experience with burnup credit*, October 1997.
16. IAEA-TECDOC-1013/5, J.-C. Neuber, *Present status and future developments of the implementation of burnup credit in spent fuel management systems in Germany*, October 1997.
17. IAEA-TECDOC-1013/6, G. Hordosy, *Burnup credit assessment in Hungary*, October 1997.
18. IAEA-TECDOC-1013/7, Y. Nomura, *Study on burnup credit evaluation method at JAERI towards securing criticality safety rationale for management of spent fuel*, October 1997.
19. IAEA-TECDOC-1013/8, H. S. Shin, Y. J. Shin, S.-G. Ro, *Application of burnup credit for spent fuel management in the Republic of Korea*, October 1997.
20. IAEA-TECDOC-1013/9, V. I. Koulikov, T. F. Makarchuk, N. S. Tikhonov, *Application of burnup credit in spent fuel management at Russian NPPs*, October 1997.
21. IAEA-TECDOC-1013/10, V. Petenyi, V. Chrapciak, *Burnup credit implementation for spent fuel management options in the Slovak Republic*, October 1997.
22. IAEA-TECDOC-1013/11, J. M. Conde, M. Recio, *Credit to fuel burnup for criticality safety evaluations in Spain*, October 1997.
23. IAEA-TECDOC-1013/12, L. Agrenius, *Burnup credit in Sweden*, October 1997.
24. IAEA-TECDOC-1013/13, L. Agrenius, *Burnup credit in the Central Storage Facility for Spent Nuclear Fuel in Sweden*, October 1997.
25. IAEA-TECDOC-1013/14, P. Grimm, *Status of burnup credit implementation in Switzerland*, October 1997.
26. IAEA-TECDOC-1013/15, R. Bowden, *The application of burnup credit for spent fuel operations in the United Kingdom*, October 1997.
27. IAEA-TECDOC-1013/16, W. Lake, *Burnup credit activities being conducted in the United States*, October 1997.
28. IAEA-TECDOC-1013/17, D. B. Lancaster et. al., *Actinide-only burnup credit methodology for PWR spent nuclear fuel*, October 1997.
29. IAEA-TECDOC-1013/18, M. C. Brady et. al., *Findings of the OECD/NEA study on burnup credit*, October 1997.

2000 Technical Committee Meeting – Vienna, 10-14 July

30. IAEA-TECDOC-1241/1, H. P. Dyck, *Implementation of burnup credit in spent fuel management systems*, July 2000.

31. IAEA-TECDOC-1241/2, M. Brady-Raap, Y. Nomura, E. Sartori, *Overview of the burnup credit activities at OECD/NEA/NSC*, July 2000.
32. IAEA-TECDOC-1241/3, R. Keqiang, X. Xiaogang, S- Leisheng, *Burnup credit study and application in spent fuel management in China*, July 2000.
33. IAEA-TECDOC-1241/4, V. Fajman, *TCM implementation of burnup credit in spent fuel management systems*, July 2000.
34. IAEA-TECDOC-1241/5, H. Toubon, *Current applications of actinide-only burnup credit within the COGEMA group and R&D programme to take fission products into account*, July 2000.
35. IAEA-TECDOC-1241/6, J.-C. Neuber, H. Kühl, *Present status and future developments of the implementation of burnup credit in spent fuel management systems in Germany*, July 2000.
36. IAEA-TECDOC-1241/7, J. M. Conde, M. Recio, *Burnup credit in Spain*, July 2000.
37. IAEA-TECDOC-1241/8, D. Mennerdahl, *Irradiated fuel storage and transport: A Swedish perspective*, July 2000.
38. IAEA-TECDOC-1241/9, P. Grimm, *Status of burnup credit implementation and research in Switzerland*, July 2000.
39. IAEA-TECDOC-1241/10, V. Medun, *Burnup credit demands for spent fuel management in Ukraine*, July 2000.
40. IAEA-TECDOC-1241/11, W. H. Lake, D. A. Thomas, T. W. Doering, *Burnup credit activities in the United States*, July 2000.
41. IAEA-TECDOC-1241/12, J.-C. Neuber, H. H. Schweer, H. G. Johann, *Regulatory status of burnup credit for dry storage and transport of spent nuclear fuel in the United States*, July 2000.
42. IAEA-TECDOC-1241/13, J. M. Conde, *Regulatory aspects of burnup credit implementation*, July 2000.
43. IAEA-TECDOC-1241/14, D. E. Carlson, *Regulatory Status of Burnup Credit for Dry Storage and Transport of Spent Nuclear Fuel in the United States*, July 2000.
44. IAEA-TECDOC-1241/15, K. Van der Meer et. al., *REBUS: A burnup credit experimental programme*, July 2000.
45. IAEA-TECDOC-1241/16, L. Markova, *Study of multiplication factor sensitivity to the spread of WWER spent fuel isotopics calculated by different codes*, July 2000.
46. IAEA-TECDOC-1241/17, U. Hesse et. al., *KENOEST – A new coupled code system based on KENO and OREST for criticality and burnup inventory calculations*, July 2000.
47. IAEA-TECDOC-1241/18, J.-C. Neuber et. al., *Siemens PWR burnup credit criticality analysis methodology: Depletion and verification methods*, July 2000.
48. IAEA-TECDOC-1241/19, V. Chrapciak, *The implementation of burnup credit in VVER-440 spent fuel*, July 2000.
49. IAEA-TECDOC-1241/20, T. W. Doering, D. A. Thomas, *Disposal criticality analysis methodology's principal isotope burnup credit*, July 2000.
50. IAEA-TECDOC-1241/21, C. V. Parks et al., *Validation Issues for Depletion and Criticality Analysis in Burnup Credit*, pp. 167-179, July 2000.
51. IAEA-TECDOC-1241/22, J.-C. Neuber, *Evaluation of axial and horizontal burnup profiles*, July 2000.
52. IAEA-TECDOC-1241/23, G. Hordosy, *Studies on future application of burnup credit in Hungary*, July 2000.

53. IAEA-TECDOC-1241/24, D. Lancaster, *Details on actinide-only burnup credit application in the USA*, July 2000.
54. IAEA-TECDOC-1241/25, C. V. Parks, M. D. DeHart, J. C. Wagner, *Phenomena and Parameters Important to Burnup Credit*, pp. 231-247, July 2000.
55. IAEA-TECDOC-1241/26, A. Lebrun, G. Bignon, *Non destructive assay of nuclear LEU spent fuels for burnup credit applications*, July 2000.
56. IAEA-TECDOC-1241/27, P. Malesys, *Transnucleaire's experience with burnup credit in transport operations*, July 2000.
57. IAEA-TECDOC-1241/28, H. G. Johann, J.-C. Neuber, *The Neckarwestheim fuel handling procedure*, July 2000.
58. IAEA-TECDOC-1241/29, Y. Nomura, K. Itahara, *Burnup credit implementation plan and preparation work at JAERI*, July 2000.
59. IAEA-TECDOC-1241/30, T. W. Doering, G. A. Cordes, *Status of the multi-detector analysis system (MDAS) and the fork detector research programs*, July 2000.
60. IAEA-TECDOC-1241/31, C. V. Parks, J. C. Wagner, *Issues for Effective Implementation of Burnup Credit*, pp. 298-308, July 2000.
61. IAEA-TECDOC-1241/32, Group discussions, *Wet storage and dry storage*, pp. 311-321, July 2000.
62. IAEA-TECDOC-1241/33, Group discussions, *Transport working group*, pp. 322-333, July 2000.
63. IAEA-TECDOC-1241/34, Group discussions, *Application of burnup credit to reprocessing*, pp. 334-336, July 2000.
64. IAEA-TECDOC-1241/35, Group discussions, *Disposal issues*, pp. 337-342, July 2000.

2002 Technical Committee Meeting – Madrid, 22-26 April

65. IAEA-TECDOC-1378/1, *Summary*, April 2002.
66. IAEA-TECDOC-1378/2, W. Danker, *Overview on the BUC activities at the IAEA*, April 2002.
67. IAEA-TECDOC-1378/3, M. Brady-Raap, *OECD/NEA report*, April 2002.
68. IAEA-TECDOC-1378/4, B. Roque, A. Santamarina, *Experimental validation of actinide and fission products inventory from chemical assays in French PWR spent fuels*, April 2002.
69. IAEA-TECDOC-1378/5, A. Courvelle et. al., *Improvement of the BUC-FP nuclear data in the JEFF library*, April 2002.
70. IAEA-TECDOC-1378/6, P. Grimm et. al., *Measurements of reactivity effects and isotopic composition of highly burnt fuel in LWR-PROTEUS Phase II*, April 2002.
71. IAEA-TECDOC-1378/7, C. Alejano, *Experimental measurement of the isotopic composition of high enrichment and high burnup fuel*, April 2002.
72. IAEA-TECDOC-1378/8, P. Baeten et. al., *The burnup credit experimental programme REBUS*, April 2002.
73. IAEA-TECDOC-1378/9, N. T. Gulliford, *BUC validation in the UK: Design of experiments and lessons learnt*, April 2002.
74. IAEA-TECDOC-1378/10, A. Machiels, A. Wells, *EPRI R&D perspective on burnup credit*, April 2002.
75. IAEA-TECDOC-1378/11, J.-C. Neuber, *Impact of the initial enrichment on the end effect*, April 2002.

76. IAEA-TECDOC-1378/12, G. Hordosy, *Studies of the influence of the spatial change of the fuel burnup on criticality in WWER-440 systems*, April 2002.
77. IAEA-TECDOC-1378/13, V. Chrapciak, P. Mikolas, *Evaluation of horizontal burnup profile for WWER-440 fuel assembly*, April 2002.
78. IAEA-TECDOC-1378/14, M. Kromar, B. Kurincic, *Burnup credit methodology in the NPP KRzko spent fuel pool reracking project*, April 2002.
79. IAEA-TECDOC-1378/15, D. Lopez, C. Töre, *WWER fuel rod isotopics by MONTEBURNS 1.0 – Influence on the multiplication factor and comparison with the CV3 benchmark data*, April 2002.
80. IAEA-TECDOC-1378/16, C. V. Parks, J. C. Wagner, I. C. Gauld, *Research to support expansion of U.S. regulatory position on burnup credit for transport and storage casks*, April 2002.
81. IAEA-TECDOC-1378/17, J.-C. Neuber, *Bounding approach to burnup credit criticality safety analysis*, April 2002.
82. IAEA-TECDOC-1378/18, Y. Kovbasenko, *Comparative analysis of multiplying properties of WWER-1000 spent fuel depending on assembly layout in the reactor core and their operating conditions*, April 2002.
83. IAEA-TECDOC-1378/19, D. Lancaster, *Practical issues with implementation of burnup credit in the USA for storage and transportation*, April 2002.
84. IAEA-TECDOC-1378/20, J.-C. Neuber, *Risk, confidence, tolerances and bias – Brief outlines of the basic concepts*, April 2002.
85. IAEA-TECDOC-1378/21, I. C. Gauld, C. V. Parks, *Strategies for applying isotopic uncertainties in burnup credit*, April 2002.
86. IAEA-TECDOC-1378/22, W. H. Lake, *Probabilistic assessment of dry transport with burnup credit*, April 2002.
87. IAEA-TECDOC-1378/23, D. N. Simister, *UK regulatory perspective on the application of burnup credit in plant criticality safety cases*, April 2002.
88. IAEA-TECDOC-1378/24, T. Doering, D. Brownson, J. Knudson, *Risk Informed Processes*, PPT presentation also available, April 2002.
89. IAEA-TECDOC-1378/25, R. Aydinyan, *Burnup factor in the licensing of Armenian NPP SFDS*, April 2002.
90. IAEA-TECDOC-1378/26, M. A. Manolova, R. I. Prodanova, T. G. Apostolov, *Criticality calculations of WWER spent fuel casks implementing burnup credit*, April 2002.
91. IAEA-TECDOC-1378/27, S. Zhao et. al., *Research and application of burnup credit technology in China*, April 2002.
92. IAEA-TECDOC-1378/28, L. Markova, J. Svarny, *Burnup credit implementation in WWER-440 dual purpose cask*, April 2002.
93. IAEA-TECDOC-1378/29, A. Miasnikov, *Computer codes qualification in the Czech Republic*, April 2002.
94. IAEA-TECDOC-1378/30, C. Lavarenne et. al., *A new method to take burnup into account in criticality studies considering an axial profile of burnup plus some fission products*, April 2002.
95. IAEA-TECDOC-1378/31, J. Vaclav, *Utilization of BUC in Slovenia*, April 2002.
96. S. H. Lee, J. G. Ahn, H. R. Hwang, *Criticality analysis with burnup credit for APR1400 in the Republic of Korea*, April 2002.
97. IAEA-TECDOC-1378/32, A. Smaizys, P. Poskas, *Possibility for BUC implementation in RBMK-1500 fuel dual purpose storage casks*, April 2002.
98. IAEA-TECDOC-1378/33, D. Mennerdahl, *BUC – A simple nuclear criticality safety concept that can be very difficult to implement*, April 2002.

99. IAEA-TECDOC-1378/34, D. Thomas et. al., *Future Disposal Burnup Credit Process and Efforts*, PPT presentation also available, April 2002.
100. IAEA-TECDOC-1378/35, B. Gmal, E. F. Moser, H. Scheib, *Consideration of burnup in criticality analysis for long term storage and final disposal of spent nuclear fuel in Germany*, April 2002.
101. IAEA-TECDOC-1378/36, H. Toubon et. al., *Burnup credit methodology for UO₂ and MOX fuel assemblies in AREVA/COGEMA*, April 2002.
102. IAEA-TECDOC-1378/37, A. Lebrun, C. Riffard, H. Toubon, *Cross-checking of the operator data used for burnup measurements*, April 2002.
103. IAEA-TECDOC-1378/38, J. Coletta, M. Brady-Raap, *Burnup credit in the evaluation of MOX fuel storage in the USA*, April 2002.
104. IAEA-TECDOC-1378/39, B. Roque, A. Santamarina, N. Thiollay, *Burnup credit calculation route for PWR MOX assemblies and experimental validation in Minerve RI-MOX and SLB1 P.I.E.*, April 2002.

2005 Technical Meeting – London, 29 August-2 September

The proceedings were not available at the end of 2005 but all papers are listed even though some of the papers were not distributed at the meeting.

105. W. Danker, *Overview of IAEA Spent Fuel Management Activities*, August 2005.
106. M. Brady-Raap, *OECD/NEA report*,
107. C. V. Parks, J. C. Wagner, *A Coordinated US Program to Address Full Burnup Credit in Transport and Storage Casks*,
108. M. D. DeHart, *Improved Radiochemical Assay Analyses Using TRITON Depletion Sequences in SCALE*,
109. T. Nakata, *Integrated Depletion Code MVP-ORBURN: Development, Validation and Application Study to the Burnup Credit Evaluation*, August 2005.
110. R. Hüggenberg, D. Winterhagen, H. Köhl, *A Burnup Credit Concept for CASTOR Transport and Storage Casks with PWR Spent Fuel*, Inte distribuerad.
111. G. Gmal, R. Kilger, J. Thiel, *Issues and Future Plans of Burnup Credit Application for Disposal*,
112. M. Brady-Raap et. al., *An Intelligent Spent Fuel Database for BWR Fuels*,
113. A. P. Chetverikov et. al., *Investigation of burnup and nuclide composition of spent nuclear fuel for use when solving “burnup credit” tasks*, August 2005.
114. B. Lance et. al., *Preliminary Analysis of the REBUS-PWR Results*,
115. M. Hennebach H. Köhl, *Monte Carlo Calculations of the REBUS Critical Experiment*,
116. D. E. Mueller, J. C. Wagner, *Application of Sensitivity/Uncertainty Methods to Burnup Credit Criticality Validation*,
117. G. Hordosy, *Investigation of the influence of the plutonium and uranium cross section uncertainties in burnup credit application*, August 2005.
118. V. Chrapciak, *Calculations of criticality and nuclide compositions for VVER-440 fuel by new version of the SCALE 5 code*, August 2005.
119. G. You et. al., *The Study of Burn-up Credit Technology for Spent Fuel Storage in China*, August 2005.
120. A. Barreau et. al., *Recent advances in French validation program and derivation of the acceptance criteria for UOx Fuel*, August 2005.
121. P. Hutt, *Development of Burn-up Credit Loading Criterion for the Sizewell B Spent Fuel Storage Ponds*, August 2005.

122. N. T. Gulliford, J. A. Edge, *Analysis of Axial Burnup Profile and Burnable Poison Loading on Spent BWR Fuel Reactivity in the THORP Dissolvers*,
123. M. Manotova et. al., *Criticality safety analysis of WWER spent fuel casks with radial burnup profile implementation*, August 2005.
124. J.-C. Neuber, *Calculation Routes to Determine Burnup Credit Loading Curves*,
125. J.-C. Neuber, W. Tippl, *Presentation of Axial and Horizontal Burnup Profiles*, August 2005.
126. J.-C. Neuber, H. G. Johann, J. Conde, *Double Contingency Principle and Prevention of Misloading Events*,
127. C. Lavarenne, J. Raby, V. Rouyer, *A Conservative Approach to Consider Burnup Credit in Criticality Studies*, August 2005.
128. L. Markova, *PBC Implementation Assessment Relating to Pool at Reactor of Dukovany NPP*, August 2005.
129. J. C. Wagner, D. E. Mueller, *Assessment of benefits for Extending Burnup Credit in Transporting PWR Spent Nuclear Fuel in the USA*,
130. G. Caplin, E. Guillou, A. Marc, *Burn-Up Credit for Receipt and Storage of UOX PWR Fuels in COGEMA/La Hague Pools*, August 2005.
131. L. Agrenius, *Swedish Nuclear Fuel and Waste Management Co – Burnup credit in the Swedish system for management of spent nuclear fuel*, August 2005.
132. J. Vaclav, *Spent Fuel Management in the Slovak Republic*, August 2005.
133. Y. Kovbasenko, *Implementation of burnup and control rod credit for storage of spent nuclear fuel in Ukraine*, August 2005.
134. R. Aydinyan, *The Possibility of Taking into Account BUC for Establishment of Licensing Requirements*,
135. C. V. Parks, C. J. Withee, *US Regulatory Recommendation for Actinide-Only Burnup Credit in Transport and Storage Casks*,
136. D. Mennerdahl, J. In de Betou, *Guide for nuclear criticality safety analysis and review – Accounting for neutron irradiation and radioactive decay*, August 2005.
137. J.-C. Neuber, *The German Burnup Credit Regulatory Standards*,
138. J.-C. Neuber, *Some Words about the 95%/95% Tolerance Limit*,
139. M. D. DeHart, M. Brady-Raap, *Group Discussion: Calculation methodology*,
140. A. Santamarina, B. Lance, *Group Discussion: Validation and criticality safety criteria*,
141. J. Gulliford, J. C. Wagner, *Group Discussion: Procedural compliance with safety criteria*,
142. C. J. Withee, V. Rouyer, *Group Discussion: Regulatory aspects in BUC*,

Other meetings related to burnup credit

1973 – IAEA Proceedings – reactor burnup physics IAEA Panel 1971 (Vienna)

143. A. M. Moncassoli-Tosi, *Burn-Up Predictions for Pressurized-Water-Reactor Cores*, July 1971.
144. B. Michelsen, *Review of the Situation in Burn-Up Physics – as seen from Denmark*, July 1971.
145. C. Hunt, *Burn-Up Physics for the Dragon High-Temperature Reactor*, July 1971.

146. V. Maly, E. Teuchert, *Separated Location of the Partially Depleted Fuel in the Pebble-Bed Reactor*, July 1971.
147. I. Pop-Jordanov, *Advances in Local Nuclear Fuel Burn-Up Physics*, July 1971.
148. R. L. Growther, *Lattice Burn-Up Calculations for Thermal Reactors*, July 1971.
149. J. G. Tyror, J. R. Askew, *The United-Kingdom Approach to the Calculation of Burn-Up in Thermal Reactors*, July 1971.
150. B. P. Rasogi, K. R. Srinivasan, A. N. Nakra, *Fuel Burn-Up Predictions for Heavy-Water-Moderated Reactors*, July 1971.
151. C. Rinaldini, *Needs and Available Calculation Tools in the Field of Homogeneous Methods for Power-Distribution and Fuel-Evolution Analysis of Overall Reactors*, July 1971.
152. A. N. Kamishan, A. N. Novikov, *Physical Characteristics and Fuel Burn-Up of Water-Moderated Water-Cooled Power Reactors*, July 1971 (in Russian).
153. C. Grant, R. Solanilla, H. Moldaschl, *Application of a Heterogeneous Method in the Simulation of Heavy-Water-Reactor Operation*, July 1971.
154. J. Griffiths, *The Nuclide Composition of the NPD Fuel*, July 1971.
155. R. L. Growther, *Lattice Burn-Up Calculations for Thermal Reactors*, July 1971.
156. A. Cricchio et. al., *Determination Experimentale du Taux de Combustion et du Contenu en Isotopes Lourds d'un Assemblage de Combustible du Reactor a Eau Bouillante de Garigliano et Comparaison avec les Calculs Theoretiques*, July 1971.
157. E. Johansson, *Some Burn-Up Experiments in Sweden*, July 1971.
158. M. Robin, J. Bouchard, *Etude Experimentale de l'Evolution du Combustible de Reacteurs a Neutrons Rapides en Fonction de l'Irradiation*, July 1971.
159. A. M. Moncassoli-Tosi et. al., *Post-Irradiated Burn-Up Analysis of Trino-Vercellese Reactor Fuel Elements. Comparison with Theoretical Results*, July 1971.
160. B. Iaponche, R. Vidal, *L'Evolution Neutronique de Combustibles en Fonction de l'Irradiation dans les Reacteurs a neutrons Thermiques*, July 1971.
161. A. J. Fudge, *A Review of Experimental Methods for the Determination of Nuclear Fuel Burn-Up*, July 1971.
162. E. Münch, *Burn-Up Determination by Irreversible Admixture of Fluence-Monitoring Nuclides to Nuclear Fuel*, July 1971.
163. H. Märkl, A. Müller, M. R. Wagner, *Special Fuel and Absorber management Problems of Light-Water Reactors and their Solution with Improved Calculation Methods*, July 1971.
164. R. L. Growther, *Lattice Burn-Up Calculations for Thermal Reactors*, July 1971.
165. Panel, *Conclusions and Recommendations of the Panel*, July 1971.

**1988 Washington D.C. Workshop on the
Use of Burnup Credit in Spent Fuel Transport Casks**

166. SAND-89-0018/1, T. L. Sanders, *Burnup Credit Issues in Spent Fuel Transportation Workshop – Overview and Objectives*, February 1988.
167. SAND-89-0018/2, G. C. Allen, *Overview of Effects of Burnup Credit on Cask Design*, February 1988.
168. SAND-89-0018/3, C. V. Parks, *Parametric Neutronic Analyses Related to Burnup Credit Cask Design*, February 1988.
169. SAND-89-0018/4, R. M. Westfall, *Effects of Burnup Credit on Cask Basket Design Spacing Requirements*, February 1988.

170. SAND-89-0018/5, R. H. Jones, *A Survey of Previous and Current Industry-Wide Efforts Regarding Burnup Credit*, February 1988.
171. SAND-89-0018/6, I. K. Hall, *Burnup Credit Effect on Proposed Cask Payloads*, February 1988.
172. SAND-89-0018/7, B. H. Wakeman, S. A. Ahmed, M. L. Smith, *Evaluation of Burnup Credit for Dry Storage Casks*, February 1988.
173. SAND-89-0018/8, J. R. Thornton, *Burnup Credit in a Dry Storage Module*, February 1988.
174. SAND-89-0018/9, L. A. Brentlinger, R. W. Peterson, P. L. Hofmann, *Analysis of Collective Life-Cycle Dose for Burnup Credit Shipping Casks*, February 1988.
175. SAND-89-0018/10, D. G. Dippold, *Burnup Credit in Nuclear Waste Transport: An Economic Analysis*, February 1988.
176. SAND-89-0018/11, R. M. Westfall, *Reactor Physics and Design Code Issues*, February 1988.
177. SAND-89-0018/12, L. E. Fischer, *An Overview of Burnup Credit Safety Issues*, February 1988.
178. SAND-89-0018/13, W. R. Lloyd, *Summary – Criticality Analysis Review Methods for Independent Spent Fuel Storage Installations*, February 1988.
179. SAND-89-0018/14, H. P. Alesso, *Summary – COG: A New Point-Wise Monte Carlo Code for Burnup Analysis*, February 1988.
180. SAND-89-0018/15, S. E. Turner, *An Uncertainty Analysis – Axial Burnup Distribution Effects*, February 1988.
181. SAND-89-0018/16, S. E. Bierman, *Feasibility of Performing Criticality Experiments with Spent LWR Fuel*, February 1988.
182. SAND-89-0018/17, C. V. Parks, *An Approach for Verifying Analysis Techniques Applicable to the Burnup Credit Technical Issue*, February 1988.
183. SAND-89-0018/18, R. H. Jones, *General Philosophy on Fuel Management for Burnup Credit Application*, February 1988.
184. SAND-89-0018/19, O. Ozer, *Overview of Fuel Management Analysis Methodology*, February 1988.
185. SAND-89-0018/20, C. T. Snow, *Nuclear Fuel Accountability Experience*, February 1988.
186. SAND-89-0018/21, R. W. Rasmussen, *Verification of Criticality Safety in On-Site Spent Fuel Storage System*, February 1988.
187. SAND-89-0018/22, D. Napolitano, *In-Core Methods for Verifying Fuel Assembly Burnup*, February 1988.
188. SAND-89-0018/23, N. P. Goldstein, *Measurements to Establish Burnup Credit for Spent Fuel Assemblies*, February 1988.
189. SAND-89-0018/24, J. T. Mihalcz, *Development of Portable Subcriticality Measurement System for Spent Fuel Shipping and Storage Casks*, February 1988.
190. SAND-89-0018/25, T. L. Sanders, *A Relative Risk Comparison of Criticality Control Strategies Based on Fresh Fuel and Burnup Credit Design Casks*, February 1988.
191. SAND-89-0018/26, W. H. Lake, *Summary Overview of the DOE Burnup Credit Workshop*, February 1988.

**1989 - INMM Spent Fuel Management Seminar VI
11-13 January 1989**

192. INMM/SFMS-VI/BUC-1, B. H. Wakeman, *Metal Cask Storage at Virginia Power*, January 1989.
193. INMM/SFMS-VI/BUC-2, N. B. McLeod, *Status of Spurces, Accuracy and Use of Spent Fuel Characteristics to Improve Design for the Disposal of Spent Fuel*, January 1989.
194. INMM/SFMS-VI/BUC-3, J. P. Roberts, *Status of Licensing of Dry Storage Technologies*, January 1989.
195. INMM/SFMS-VI/BUC-4, B. H. Wakeman, *Criticality Measurements and Their Application to Spent Fuel Storage*, January 1989.

**1992 - INMM Spent Fuel Management Seminar IX
Special Burnup Credit Session,
Washington D.C., 21-22 February 1992**

196. INMM/SFMS-IX/BUC-1, T. L. Sanders, K. D. Seager, R. I. Ewing, *Overview of Burnup Credit Issues*, February 1988.
197. INMM/SFMS-IX/BUC-2, R. I. Ewing, S. R. Bierman, *Measurement Techniques for Verifying Burnup*, February 1988.
198. INMM/SFMS-IX/BUC-3, M. C. Brady, D. G. Napolitano, *Validation Analysis Methodologies Used in Burnup Credit Criticality Calculations*, February 1988.
199. INMM/SFMS-IX/BUC-4, T. L. Sanders, K. D. Seager, R. I. Ewing, *Strategies for Certifying a Burnup Credit Cask*, February 1988.
200. INMM/SFMS-IX/BUC-5, C. R. Marotta, M. C. Brady, D. G. Napolitano, *Effect of Axial Exposure Distributions in Burnup Credit in Burnup Credit Criticality Analyses*, February 1988.

2000 Burnup Credit PIRT Activities, NRC, Washington D.C.,

201. NUREG/CR-6764, G. H. Bidinger et al., *Burnup Credit PIRT Report*, May 2002.

**2002 – ANS Tutorial - Burnup Credit
ANS Annual Meeting 2002 (Hollywood, Florida)**

202. Burnup Credit Tutorial Session, M. Brady-Raap, *Burnup Credit Activities at OECD/NEA/NSC and IAEA*, June 2002.
203. Burnup Credit Tutorial Session, D. Diamond, *Report of Expert panel on Burnup Credit*, June 2002.
204. Burnup Credit Tutorial Session, M. DeHart, *Rapid Depletion Calculations Using ORIGEN-ARP*, June 2002.
205. Burnup Credit Tutorial Session, M. DeHart, *Experimental Data and Programs Useful for Burnup Credit*, June 2002.
206. Burnup Credit Tutorial Session, M. DeHart, *Radiochemical Assay Data for Use in Burnup Credit*, June 2002.
207. Burnup Credit Tutorial Session, C. Parks, *Review of NRC Guidance on Burnup Credit for Transport and Dry Cask Storage*, June 2002.

208. Burnup Credit Tutorial Session, J. Wagner, *Demonstration of Approach for Developing BUC Loading Curve(s)*, June 2002.
209. ORNL, *Burnup Credit Related Publications*, June 2002.

National and international standards

210. ANSI/ANS-8.27, Draft, *Burnup Credit for LWR*, June 2006. This is not yet publicly available but has been submitted to ANS for review.
211. DIN 25712 Draft, *Criticality Safety with Fuel Burnup Credit in the Transport and Storage of Irradiated Light-Water Reactor Fuel Assemblies in Casks*, July 2003.
212. DIN 25471, *Criticality Safety with Fuel Assembly Burnup Credit in the Storage and Handling of Fuel Assemblies in Fuel Assembly Storage Pools of Nuclear Power Plants with Light-Water Reactors*, September 2000.
213. KTA 3602, *Storage and Handling of Fuel Assemblies and Associated Equipment in Nuclear Power Plants with Light-Water Reactors*, November 2003.
214. UNE 73-501-92, Norma Española, *Requisitos de criticidad para el diseño de bastidores de almacenamiento en piscinas de combustible*, November 1991.

ORNL

215. ORNL-6698 (NUREG/C R-5625), O. W. Hermann et. al, *Technical Support for a Proposed Decay Heat Guide Using SAS2H/ORIGEN-S Data*. July 1994.
216. ORNL/CP-97228. See PATRAM 1998, Shappert et. al.
217. ORNL/CSD/TM-244, S. P. Cerne, O. W. Hermann, R. M. Westfall, *Reactivity and Isotopic Composition of Spent PWR Fuel as a Function of Initial Enrichment, Burnup, and Cooling Time*, October 1987.
218. ORNL/M-1073, J.-P. Renier, C. V. Parks, *Executive Summary: Reactor Critical Benchmark Calculations for Burnup Credit Applications*, April 1990.
219. ORNL/M-1423 Draft, S. M. Bowman, *Criticality Reference Benchmark Calculations for Burnup Credit Using Spent Fuel Isotopics*, April 1991.
220. ORNL-M-5003, L. B. Shappert (Managing Editor), *The Radioactive Materials Packaging Handbook – Design, Operations, and Maintenance*,
221. ORNL-M-6121, O. W. Hermann, M. D. DeHart, B. D. Murphy, *Evaluation of Measured LWR Spent Fuel Composition Data for Use in Code Validation End-User Manual*, February 1998.
222. ORNL/M-6155, B. L. Broadhead, *K-infinite Trends with Burnup Enrichment, and Cooling Time for BWR Fuel Assemblies*, August 1998.
223. ORNL-NSIC-68, L. B. Shappert, *Cask Designer's Guide*, February 1970.
224. ORNL/TM-6889, B. D. Murphy, T. D. Newton, S. Raman, *Preliminary Computational Analysis of the Actinide Samples from FP-4 Exposed in the Dounreay Prototype Fast Reactor*. December 1996.
225. ORNL/TM-6901. Se NEA/NSC/DOC(96)/06 under OECD/NEA
226. ORNL/TM-10404, A. D. Kelmers et. al., *Identification and Evaluation of Radionuclide Generation/Depletion Codes for Potential Use by the Department of Energy's Office of Civilian Radioactive Waste Management*, February 1989.
227. ORNL/TM-12294/V1, M. D. DeHart, *SCALE-4 Analysis of Pressurized Water Reactor Critical Configurations: Vol. 1 – Summary*, March 1995.

228. ORNL/TM-12294/V2, S. M. Bowman, O. W. Hermann, M. C. Brady, *SCALE-4 Analysis of Pressurized Water Reactor Critical Configurations: Vol. 2 – Sequoyah Unit 2 Cycle 3*, March 1995.
229. ORNL/TM-12294 Vol. 3, S. M. Bowman, O. W. Hermann, *SCALE-4 Analysis of Pressurized Water Reactor Critical Configurations: Vol. 3 – Surry Unit 1 Cycle 2*, March 1995.
230. ORNL/TM-12294/V4, M. D. DeHart, *SCALE-4 Analysis of Pressurized Water Reactor Critical Configurations: Vol. 4 – Three Mile Island Unit 1 Cycle 5*, March 1995.
231. ORNL/TM-12294/V5, S. M. Bowman, O. W. Suto, *SCALE-4 Analysis of Pressurized Water Reactor Critical Configurations: Vol. 5 – North Anna Unit 1 Cycle 5*, October 1996.
232. ORNL/TM-12667, O. W. Hermann et. al, *Validation of the SCALE System for PWR Spent Fuel Isotopic Composition Analysis*. March 1995.
233. ORNL/TM-12742, B. L. Broadhead, et. al., *Investigation of Nuclide Importance to Functional Requirements Related to Transport and Long-Term Storage of LWR Spent Fuel*, June 1995.
234. ORNL/TM-12959, M. D. DeHart, S. M. Bowman, *Analysis of Fresh Fuel Criticality Experiments Appropriate for Burnup Credit Validation*, October 1995.
235. ORNL/TM-12973, M. D. DeHart, *Sensitivity and Parametric Evaluations of Significant Aspects of Burnup Credit for PWR Spent Fuel Packages*, May 1996.
236. ORNL/TM-13170/V1, B. D. Murphy, *Characteristics of Spent Fuel from Plutonium Disposition Reactors – Vol. 1: The Combustion Engineering System 80+ Pressurized-Water-Reactor Design*, June 1996.
237. ORNL/TM-13170/V2, J. C. Ryman, O. W. Hermann, *Characteristics of Spent Fuel from Plutonium Disposition Reactors – Vol. 2: A General Electric Boiling-Water-Reactor Design*, April 1998.
238. ORNL/TM-13170/V3, B. D. Murphy, *Characteristics of Spent Fuel from Plutonium Disposition Reactors – Vol. 3: A Westinghouse Pressurized-Water-Reactor Design*, July 1997.
239. ORNL/TM-13170/V4, B. D. Murphy, *Characteristics of Spent Fuel from Plutonium Disposition Reactors – Vol. 4: A Westinghouse Pressurized-Water-Reactor Fuel Cycle Without Integral Absorber*, April 1998.
240. ORNL/TM-13315, O. W. Hermann, M. D. DeHart, *Validation of SCALE/(SAS2H) Isotopic Predictions for BWR Spent Fuel*, September 1998.
241. ORNL/TM-13317, M. D. DeHart, O. W. Hermann, *An Extension of the Validation of SCALE/(SAS2H) Isotopic Predictions for PWR Spent Fuel*, September 1996.
242. ORNL/TM-13422, S. E. Fisher et. al., *Nuclear Data to Support Computer Code Validation*, April 1997.
243. ORNL/TM-13687, B. D. Murphy, *Prediction of Isotopic Composition of UO₂ Fuel from a BWR: Analysis of the DU1 Sample from the Dodewaard Reactor*. October 1998.
244. ORNL/TM-1999/99, M. D. DeHart, *Parametric Analysis of PWR Spent Fuel Depletion Parameters for Long-Term Disposal Criticality Safety*, August 1999.
245. ORNL/TM-1999/101, C. V. Parks et. al., *Annual EMSP Summary Progress Report, Project Title: Development of Nuclear Analysis Capabilities for DOE Waste Management Activities*, June 1999.
246. ORNL/TM-108/R1, O. W. Hermann, *San Onofre PWR Data for Code Validation of MOX Fuel Depletion Analyses*, March 2000.

247. ORNL/TM-1999/168, B. D. Murphy et. al., *Simulation of Low-Enriched Uranium (LEU) Burnup in Russian VVER Reactors with the HELIOS Code Package*, March 2000.
248. ORNL/TM-1999/193, J. C. Wagner, M. D. DeHart, B. L. Broadhead, *Investigation of Burnup Credit Modeling Issues Associated with BWR Fuel*, October 2000.
249. ORNL/TM-1999/246, J. C. Wagner, M. D. DeHart, *Review of Axial Burnup Distribution Considerations for Burnup Credit Calculations*, March 2000.
250. ORNL/TM-1999/247, I. C. Gauld, *SCALE-4 Analysis of LaSalle Unit 1 BWR Commercial Reactor Critical Configurations*, March 2000.
251. ORNL/TM-1999/303, (NUREG/CR-6665), C. V. Parks, M. D. DeHart, J. C. Wagner, *Review and Prioritization of Technical Issues Related to Burnup Credit for LWR Fuels*, February 2000.
252. ORNL/TM-1999/326, O. W. Hermann, *Benchmark of SCALE (SAS2H) Isotopic Predictions of Depletion Analyses for San Onofre PWR MOX Fuel*, February 2000.
253. ORNL/TM-2000/72 (NUREG/CR-6702), I. C. Gauld, *Limited Burnup Credit in Criticality Safety Analysis: A Comparison of ISG-8 and Current International Practice*, January 2001.
254. ORNL/TM-2000/230 (NUREG/CR-6683), J. C. Wagner, C. V. Parks, *A Critical Review of the Practice of Equating the Reactivity of Spent Fuel to Fresh Fuel in Burnup Credit Safety Analyses for PWR Spent-Fuel Pool Storage*, September 2000.
255. ORNL/TM-2000/277 (NUREG/CR-6701), I. C. Gauld, C. V. Parks, *Review of Technical Issues Related to Predicting Isotopic Compositions and Source Terms for High-Burnup LWR Fuel*, January 2001.
256. ORNL/TM-2000/284 (NUREG/CR-6700), I. C. Gauld, J. C. Ryman, *Nuclide Importance to Criticality Safety, Decay Heating and Source Terms Related to Transport and Interim Storage of High-Burnup LWR Fuel*, January 2001.
257. ORNL/TM-2000/306 (NUREG/CR-6747), J. C. Wagner, *Computational Benchmark for Estimation of Reactivity Margin from Fission Products and Minor Actinides in PWR Burnup Credit*, October 2001.
258. ORNL/TM-2000/321 (NUREG/CR-6760), C. E. Sanders, J. C. Wagner, *Study of the Effect of Integral Burnable Absorbers for PWR Burnup Credit*, March 2002.
259. ORNL/TM-2000/373 (NUREG/CR-6761), J. C. Wagner, C. V. Parks, *Parametric Study of the Effect of Burnable Poison Rods for PWR Burnup Credit*, March 2002.
260. ORNL/TM-2000/385 (NUREG/CR-6716), S. M. Bowman, I. C. Gauld, J. C. Wagner, *Recommendation on Fuel Parameters Standard Technical Specifications for Fuel Storage Casks*, March 2001.
261. ORNL/TM-2001/50, C. E. Sanders, R. T. Primm III, *Quality Assurance Calculations to Support Use of HELIOS Version 1.6 for Plutonium Disposition Studies*, April 2001.
262. ORNL/TM-2001/33 (NUREG/CR-6748), I. C. Gauld, S. M. Bowman, *STARBUCS: A Prototypic SCALE Control Module for Automated Criticality Safety Analyses Using Burnup Credit*, October 2001.
263. ORNL/TM-2001/69 (NUREG/CR-6759), C. E. Sanders, J. C. Wagner, *Parametric Study of the Effect of Control Rods for PWR Burnup Credit*, February 2002.
264. ORNL/TM-2001/83, M. D. DeHart, *A Stochastic Method for Estimating the Effect of Isotopic Uncertainties in Spent Nuclear Fuel*, September 2001.

265. ORNL/TM-2001/200, M. E. Dunn, *ORNL Support for Yucca Mountain Project, Fiscal Year 2001 Status Report for the Investigation of Reactivity Effects Due to Perturbations in Cross-Section Temperatures*, September 2001.
266. ORNL/TM-2001/257 (NUREG/CR-6811), I. C. Gauld, *Strategies for Applications of Isotopic Uncertainties in Burnup Credit*, June 2003.
267. ORNL/TM-2001/259 (NUREG/CR-6798), C. E. Sanders, I. C. Gauld, *Isotopic Analysis of High-Burnup PWR Spent Fuel Samples From the Takahama-3 Reactor*, January 2003
268. ORNL/TM-2001/272 (NUREG/CR-6781), J. C. Wagner, C. V. Parks, *Recommendations on the Credit for Cooling Time in PWR Burnup Credit Analyses*, January 2003.
269. ORNL/TM-2001/273 (NUREG/CR-6801), J. C. Wagner, M. D. DeHart, C. V. Parks, *Recommendations for Addressing Axial Burnup in PWR Burnup Credit Analyses*, March 2003.
270. ORNL/TM-2002/6 (NUREG/CR-6800), J. C. Wagner, C. E. Sanders, *Assessment of Reactivity Margins and Loading Curves for PWR Burnup Credit Cask Designs*, March 2003.
271. ORNL/TM-2002/110, C. V. Parks et. al., *Plutonium Production Using Natural Uranium From the Front-End of the Nuclear Fuel Cycle*, September 2002.
272. ORNL/TM-2002/240, C. M. Hopper et. al., *Design Parameters for a Natural Uranium UO_3 - or U_3O_8 -Fueled Nuclear Reactor*, November 2002.
273. ORNL/TM-2002/255 (NUREG/CR-6835), K. R. Elam et. al., *Effects of Fuel Failure on Criticality Safety and Radiation Dose for Spent Fuel Casks*, September 2003.
274. ORNL/TM-2005/48, I. C. Gauld, D. E. Mueller, *Evaluation of Cross-Section Sensitivities in Computing Burnup Credit Fission Product Concentrations*, August 2005.
275. W6479/IIR-01-01, J. C. Wagner, C. V. Parks, I. C. Gauld, *Technical Bases to Support Recommendations and Proposed Guidance for Expansion of ISG-8, Revision 1*, February 2001 (förvaras med NRC ISG-8).

SCALE and burnup credit

276. M. D. DeHart, Z. Zhong, T. J. Downar, *TRITON: An Advanced Lattice Code for MOX Fuel Calculations*, *Advances in Nuclear Fuel Cycle Management III*, October 2003.
277. ORNL/TM-2003/2, I. C. Gauld, *MOX Cross-Section Libraries for ORIGEN-ARP*, July 2003.
278. ORNL/TM-2003/263, B. D. Murphy, *ORIGEN-ARP Cross-Section Libraries for Magnox, Advanced Gas-Cooled, and VVER Reactor Designs*, February 2004.
279. ORNL/TM-2005/39-V5, Vol I, Book 2, Sect. C8, B. T. Rearden, *TSUNAMI-1D: Control Module for One-Dimensional Cross-Section Sensitivity and Uncertainty Analysis for Criticality*, April 2005.
280. ORNL/TM-2005/39-V5, Vol I, Book 2, Sect. C9, B. T. Rearden, *TSUNAMI-3D: Control Module for Three-Dimensional Cross-Section Sensitivity and Uncertainty Analysis for Criticality*, April 2005.
281. ORNL/TM-2005/39-V5, Vol I, Book 2, Sect. C10, I. C. Gauld, S. M. Bowman, *STARBUCS: A SCALE Control Module for Automated Criticality Analysis Using Burnup Credit*, April 2005.

282. ORNL/TM-2005/39-V5, Vol I, Book 2, Sect. D1, I. C. Gauld, et. al., *ORIGEN-ARP: Automatic Rapid Processing for Spent Fuel Depletion, Decay, and Source Term Analysis*, April 2005.
283. ORNL/TM-2005/39-V5, Vol I, Book 3, Sect. S2, I. C. Gauld, O. W. Hermann, *SAS2: A Coupled One-Dimensional Depletion and Shielding Analysis Module*, April 2005.
284. ORNL/TM-2005/39-V5, Vol I, Book 3, Sect. T1, M. D. DeHart, *TRITON: A Two-Dimensional Depletion Sequence for Characterization of Spent Nuclear Fuel*, April 2005.
285. ORNL/TM-2005/39-V5, Vol II, Book 1, Sect. F7, I. C. Gauld, O. W. Hermann, R. M. Westfall, *ORIGEN-S: SCALE System Module to Calculate Fuel Depletion, Actinide Transmutation, Fission Product Buildup and Decay, and Associated Radiation Source Terms*, April 2005.
286. ORNL/TM-2005/39-V5, Vol II, Book 4, Sect. F21, M. D. DeHart, *NEWT: A New Transport Algorithm for Two-Dimensional Discrete Ordinates Analysis in Non-Orthogonal Geometries*, April 2005.
287. ORNL/TM-2005/39-V5, Vol III, Sect. M18, B. T. Rearden, *Sensitivity Utility Modules*, April 2005.

NRC

288. ASLBP No. 99-762-02-LA, NRC Atomic Safety and Licensing Board, *Deposition of Laurence I. Kopp November 21 1999*, January 2000. Carolina Power & Light CO. Shearon Harris DOCKET No. 50-400-LA.
289. IN 92-21, Information Notice, *Spent Fuel Pool Calculations*, March 1992.
290. J. R. Strosnider (NRC Office of Nuclear Material Safety) letter to C. J. Paperiello (NRC Office of Nuclear Regulatory Research), *User Need Memorandum – Request for Assistance Regarding Extension of the Technical Basis for Burnup Credit in Spent Fuel Casks*, 11 May 2005.
291. L. Kopp in attachment to letter to T. Collins, *Guidance on the Regulatory Requirements for Criticality Analysis of Fuel Storage at Light-Water Reactor Power Plants*, 19 August 1998.
292. NUREG-1520, *Standard Review Plan for a Fuel Cycle Facility*, March 2002.
293. NUREG-1536, *Standard Review Plan for Dry Cask Storage Systems*, January 1997.
294. NUREG-1617 Suppl. 1, *Standard Review Plan for Transportation Packages for MOX Spent Nuclear Fuel*, September 2005.
295. NUREG-1617, *Standard Review Plan for Transportation Packages for Spent Nuclear Fuel*, March 2000.
296. NUREG-1804 Rev 2, *Yucca Mountain Review Plan, Final Report*, CNWRA, July 2003.
297. NUREG/CR-5625. Se ORNL-6698.
298. NUREG/CR-6665. Se ORNL/TM-1999/303.
299. NUREG/CR-6683. Se ORNL/TM-2000/230.
300. NUREG/CR-6700. Se ORNL/TM-2000/284.
301. NUREG/CR-6701. Se ORNL/TM-2000/277.
302. NUREG/CR-6702. Se ORNL/TM-2000/72.
303. NUREG/CR-6716. Se ORNL/TM-2000/385.
304. NUREG/CR-6748. Se ORNL/TM-2001/33.
305. NUREG/CR-6759. Se ORNL/TM-2001/69.

306. NUREG/CR-6760. Se ORNL/TM-2000/321.
307. NUREG/CR-6761. Se ORNL/TM-2000/373.
308. NUREG/CR-6781. Se ORNL/TM-2001/272.
309. NUREG/CR-6798. Se ORNL/TM-2001/259.
310. NUREG/CR-6800. Se ORNL/TM-2002/6.
311. NUREG/CR-6801. Se ORNL/TM-2001/273.
312. NUREG/CR-6811. Se ORNL/TM-2001/257.
313. NUREG/CR-6835. Se ORNL/TM-2002/255.
314. RG 1.13 Rev 2 Draft, Regulatory Guide 1.13, *Spent Fuel Storage Facility Design Basis*, December 1981.
315. RIL-178, Research Information Letter, *Burnup Credit for Transport and Dry Cask Storage of PWR Spent Nuclear Fuel*, February 2001.
316. RIS 2001-12, Regulatory Issue Summary, *Nonconservatism in Pressurized Water Reactor Spent Fuel Storage Pool Reactivity Equivalencing Calculations*, May 2001.
317. RIS 2005-05, Regulatory Issue Summary, *Regulatory Issues Regarding Criticality Analyses for Spent Fuel Pools and Independent Spent Fuel Storage Installations*, March 2005.
318. S. Dembek till H. A. Sepp., *Non-Conservatism in Axial Burnup Biases for Spent Fuel Rack Criticality Analysis Methodology*, August 2001.
319. *Safety Evaluation Report for Disposal Criticality Analysis Methodology Topical Report*, Revision 0, June 2000.
320. SFPO ISG-8, Spent Fuel Project Office Interim Staff Guidance – 8, *Limited Burnup Credit*, NRC, May 1999.
321. SFPO ISG-8 Rev 1, Spent Fuel Project Office Interim Staff Guidance – 8, Rev.1, *Limited Burnup Credit*, NRC, July 1999.
322. SFPO ISG-8 Rev 2, Spent Fuel Project Office Interim Staff Guidance – 8, Rev.2, *Limited Burnup Credit*, NRC, September 2002.
323. TAC No. M93254, *Acceptance for Referencing of Licensing Topical Report WCAP-14416-P*, October 1996. Included in WCAP-14416-AP Rev. 1 (See Westinghouse report).
324. *Technical Study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants*, Draft, February 2000.
325. WASH-1142, G. M. Inman, *Reactor Fuel Burnup Calculations, An Annotated Bibliography of Selected Literature*, October 1969.
326. NRC/NEI/Workshop-1999, *Spent Fuel Cask Generic Issues*, December 1999.

Licensing of fuel storage in the U.S.A.

There is an extensive amount of documents related to licensing of fuel storage. Many of those include some kind of burnup credit. The following documents are important to understand some of the licensing problems in the U.S.A. (and elsewhere).

327. Westinghouse/NSAL-00-015, Westinghouse Nuclear Safety Advisory Letter, *Axial Burnup Shape Reactivity Bias*, November 2000. Warning on non-conservatism in WCAP-14416-NP-A
328. WCAP-14416-NP-A Rev. 1, W. D. Newmyer, *Westinghouse Spent Fuel Rack Criticality Analysis Methodology*, November 1996 Revision 1. Approval by NRC and other communication included in the report.

Yucca Mountain Project

329. ANL-EBS-NU-000008 Rev 01, *Screening Analysis of Criticality Features, Events, and Processes for License Application*, J. A. McClure, October 2004.
330. *Bounding Axial Profile Analysis for the Topical Report Database*, T. A. Parish, C. H. Chen, May 1997.
331. CAL-DS0-NU-000002 Rev 00B, *Configuration Generator Model*, A. Alsaed, September 2003.
332. CAL-DS0-NU-000003 Rev 00A, *Criticality Model*, A. Alsaed, September 2003.
333. CAL-DSD-NU-000003 Rev 00A, *Analysis of Critical Benchmark Experiments and Critical Limit Calculations for DOE SNF*, D. R. Moscalu, M. Saglam, July 2003.
334. CAL-DSU-NU-000001 Rev A, *Calculation of Isotopic Bias and Uncertainty for PWR Spent Nuclear Fuel*, G. Radulescu, Ersätter CAL-UDC-NU-000012 Rev A.
335. CAL-DSU-NU-000004 Rev 00A, *Isotopic Generation and Confirmation of the PWR Application Model*, L. B. Wimmer, November 2003.
336. CAL-DSU-NU-000005 Rev 00A, *BWR Axial Profile*, August 2005.
337. CAL-DSU-NU-000006 Rev 00C, *21-PWR Waste Package with Absorber Plates Loading Curve Evaluation*, J. M. Scaglione, December 2004.
338. CAL-DSU-NU-000007 Rev 00B, *Isotopic Model for Commercial SNF Burnup Credit*, J. M. Scaglione, November 2004.
339. CAL-DSU-NU-000008 Rev 00A, *44-BWR Waste Package with Loading Curve Evaluation*, J. M. Scaglione, August 2004.
340. CAL-DSU-NU-000012 Rev 00A, *PWR Axial Burnup Profile Analysis*, J. M. Scaglione, September 2003.
341. CAL-EBS-NU-000007 Rev 00, *Criticality Probability for MOX SNF*, P. Gottlieb, September 1999.
342. CAL-EBS-NU-000008 Rev 00, *Criticality Consequence Calculation Involving Intact PWR MOX SNF in a Degraded 21 PWR Assembly Waste Package*, J. A. McLure, September 1999.
343. CAL-EBS-NU-000015 Rev 00, *External Radioaction Effects from Criticality of a Degraded Commercial Waste Package*, J. S. Tang, May 2002.
344. CAL-EBS-NU-000017 Rev 00, *Radiolytic Source Generation from Internal Waste package*, J. A. McLure, G. Radulescu, September 2001.
345. CAL-EDC-NU-000008 Rev 00, *Benchmark and Critical Limit Calculation for DOE SNF*, J. M. Scaglione, March 2002.
346. CAL-UDC-NU-000006 Rev 00B, *PWR Assembly End-Effect Evaluation*, J. M. Scaglione, J. Monroe-Rammsy, March 2001.
347. CAL-UDC-NU-000008 Rev 00, *21 PWR Waste Package Variable Spent Nuclear Fuel Loading Evaluation*, J. M. Scaglione, September 2001.
348. CAL-UDC-NU-000009 Rev 00A, *PWR Depletion Parameter Sensitivity Evaluation*, J. M. Scaglione, J. Monroe-Rammsy, September 2001.
349. CAL-UDC-NU-000011 Rev A, *Three Mile Island Unit 1 Radiochemical Assay Comparisons to SAS2H Calculations*, J. M. Scaglione, April 2002.
350. CAL-UDC-NU-000012 Rev A, *Calculation of Isotopic Bias and Uncertainty for PWR Spent Nuclear Fuel*, G. Radulescu, Kommentar June 2002. Ersatt av CAL-DSU-NU-000001 Rev A.
351. CAL-WHS-MD-000001, *Waste Package Misload Probability*, J. K. Knudsen, November 2001.

352. CAL-WHS-MD-000003, *Commercial Spent Nuclear Fuel Waste Package Method Analysis*, J. K. Knudsen, September 2003.
353. CNWRA 95-011, *Compliance Determination Computer Codes for Preclosure Safety Assessments*, H. Karimi et. al., June 1995.
354. DC # 29273, *Spent Fuel Criticality Benchmark Experiments*, J. M. Scaglione, July 2001.
355. DC # 33504, *Isotopic Bias and Uncertainty for Burnup Credit Applications*, J. M. Scaglione, August 2002.
356. DOE/RW-0492 Rev 2, *Topical Report on Actinide-Only Burnup Credit for PWR Spent Nuclear Fuel Packages*, September 1998.
357. DOE/RW-0495, *Depletion and Package Modeling Assumptions for PWR Actinide-Only Burnup Credit*, C. Kang, D. Lancaster, May 1997.
358. DOE/RW-0496, *Horizontal Burnup Gradient Datafile for PWR Assemblies*, M. Rahimi, E. Fuentes, D. Lancaster, May 1997.
359. DOE/RW-0497, *Isotopic and Criticality Validation for PWR Actinide-Only Burnup Credit*, C. Kang, K. Waldrop, J. Thornton, May 1997.
360. Enclosure to CCS # 0112054425, *Source of Burnup Values for CSNF Assemblies*, Bechtel SAIC, December 2004.
361. Framatome ANP DI 32-5029773-00, *Critical Limit Development for 21 PWR Waste Package*, J. Sapyta, September 2003.
362. Framatome ANP DI 32-503964-00, *Critical Limit Development for PWR and BWR Waste Package*, J. Sapyta, December 2003.
363. Framatome DC # 24288, *Overview of BWR Fuel Isotopic Predictions Using SAS2H*, P. M. O' Leary, May 2000.
364. LA-UR-04-3482, *ENDF/B Information Review in Support of the Yucca Mountain Criticality Safety Methodology*, D. G. Bowen, S. P. Monahan, May 2004.
365. OCRWM 30033-2003 Rev. 01, *Software Qualification Report for MCNP Version 4B2, A General Monte Carlo N-Particle Transport Code*, S. Goluoglu, April 1998.
366. OCRWM Draft White Paper on Burnup Measurements, Tom Doering email, February 2002.
367. OCRWM 100-00C-WHS0-00100-000-00C, *Dry Transfer Facility Criticality Safety Evaluations*, C. E. Sanders, May 2005.
368. OCRWM 140-00C-HCR0-00300-000-00B, *Transportation Cask Receipt/Return Facility Criticality Safety Evaluations*, C. E. Sanders, April 2005.
369. OCRWM 170-00C-HA00-00100-000-00B, *Aging Facility Criticality Safety Evaluations*, C. E. Sanders, September 2004.
370. OCRWM 190-00C-CH00-00100-000-00B, *Canister Handling Facility Criticality Safety Calculations*, C. E. Sanders, April 2005.
371. OCRWM 210-00C-FH00-00400-000-00A, *Fuel Handling Facility Criticality Safety Calculations*, C. E. Sanders, June 2004.
372. OCRWM B0000000-01717-0200-00138 Rev 00B, *SAS2H Analysis of Radiochemical Assay Samples From Calvert Cliffs PWR Reactor*, M. Nichol, October 1999.
373. OCRWM B0000000-01717-0200-00139 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From Obrigheim PWR Reactor*, M. Nichol, September 1997.
374. OCRWM B0000000-01717-0200-00140 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From H. B. Robinson PWR Reactor*, M. Nichol, September 1997.

375. OCRWM B0000000-01717-0200-00141 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From Turkey Point PWR Reactor*, M. Nichol, September 1997.
376. OCRWM B0000000-01717-0200-00142 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From Trino Vercelles PWR Reactor*, M. Nichol, September 1997.
377. OCRWM B0000000-01717-0200-00143 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From Yankee Rowe PWR Reactor*, M. Nichol, September 1997.
378. OCRWM B0000000-01717-0200-00144 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From Mihama PWR Reactor*, M. Nichol, September 1997.
379. OCRWM B0000000-01717-0200-00145 Rev 00, *SAS2H Analysis of Radiochemical Assay Samples From Cooper BWR Reactor*, M. Nichol, September 1997.
380. OCRWM B0000000-01717-0200-00146 Rev 00, *MCNP CRC Reactivity Calculations for Quad Cities Unit 2*, D. P. Henderson, September 1997.
381. OCRWM B0000000-01717-0210-00001 Rev 00A, *CRC Depletion Calculations for Crystal River Unit 3*, J. M. Scaglione, June 1998.
382. OCRWM B0000000-01717-0210-00002 Rev 00A, *CRC Reactivity Calculations for Crystal River Unit 3*, J. M. Scaglione, June 1998.
383. OCRWM B0000000-01717-0210-00003 Rev 00, *CRC Depletion Calculations for McGuire Unit 1*, K. D. Wright, April 1998.
384. OCRWM B0000000-01717-0210-00004 Rev 00A, *CRC Reactivity Calculations for McGuire Unit 1*, K. D. Wright, August 1998.
385. OCRWM B0000000-01717-0210-00005 Rev 00A, *CRC Depletion Calculations for Sequoyah Unit 2*, J. M. Scaglione, May 1998.
386. OCRWM B0000000-01717-0210-00006 Rev 00A, *CRC Reactivity Calculations for Sequoyah Unit 2*, J. M. Scaglione, August 1998.
387. OCRWM B0000000-01717-0210-00007 Rev 00A, *CRC Depletion Calculations for the Three Mile Island Unit 1*, M. K. Au-Yang, May 1998.
388. OCRWM B0000000-01717-0210-00008 Rev 00A, *CRC Reactivity Calculations for the Three Mile Island Unit 1*, K. D. Wright, J. M. Scaglione, April 1998.
389. OCRWM B0000000-01717-0210-00009 Rev 01, *CRC Depletion Calculations for Quad Cities Unit 2*, D. P. Henderson, September 1999.
390. OCRWM B0000000-01717-0210-00010 Rev 01, *CRC Reactivity Calculations for Quad Cities Unit 2*, W. J. Anderson, August 1998.
391. OCRWM B0000000-01717-0210-00018 Rev 01A, *Laboratory Critical Experiments Calculations*, J. M. Scaglione, June 1999.
392. OCRWM B0000000-01717-0210-00028 Rev 1, *Range of Neutronic Parameters Calculation File*, W. J. Anderson, September 1999.
393. OCRWM B0000000-01717-0210-00034 Rev 00A, *LCE for Research Reactor Benchmark Calculations*, J. M. Scaglione, L. Angers, December 1998.
394. OCRWM B0000000-01717-0210-00041 Rev 00, *Waste Package Criticality Control Parametric Analyses*, J. M. Scaglione, February 1999.
395. OCRWM B0000000-01717-0210-00043 Rev 00, *Rod Consolidation Waste Package Criticality Calculations*, J. M. Scaglione, March 1999.
396. OCRWM B0000000-01717-0210-00097 Rev 00, *CRC Depletion Calculations for LaSalle Unit 1*, D. P. Henderson, October 1999.

397. OCRWM B0000000-01717-0210-00106 Rev 00, *The Effect of Half-Life and Branching Fraction Uncertainties on the Effective Neutron Multiplication Factor*, C. W. Connell, R. A. Kichendarfer, September 1999.
398. OCRWM B0000000-01717-0210-00107 Rev 00, *Waste Package, LCE, CRC and, Radiochemical Assay Comparison Evaluation*, J. M. Scaglione, August 1999.
399. OCRWM B0000000-01717-5705-00060 Rev 01, *Summary Report of Commercial Reactor Critical Analyses Performed for Crystal River Unit 3*, C. W. Mays, August 1998. Replaced!
400. OCRWM B0000000-01717-5705-00063 Rev 01, *Summary Report of Commercial Reactor Critical Analyses Performed for McGuire Unit 1*, C. W. Mays. April 1998.
401. OCRWM B0000000-01717-5705-00064 Rev 01, *Summary Report of Commercial Reactor Critical Analyses Performed for Sequoyah Unit 2*, K. D. Wright, C. W. Mays, April 1998.
402. OCRWM B0000000-01717-5705-00067 Rev 00, *Summary Report of Commercial Reactor Critical Analyses Performed for Catawba Unit 1*, C. W. Mays, April 1998.
403. OCRWM B0000000-01717-5705-00070 Rev 00, *Summary Report of Commercial Reactor Critical Analyses Performed for Davis-Besse Unit 1*, C. W. Mays, April 1998.
404. OCRWM B0000000-01717-5705-00075 Rev 01, *Summary Report of Commercial Reactor Critical Analyses Performed for the Disposal Criticality Analysis Methodology*, D. A. Thomas, August 1998.
405. OCRWM B0000000-01717-5705-00076 Rev 02, *Summary Record of Laboratory Critical Experiment Analyses performed for the Disposal Criticality Analysis Methodology*, J. M. Scaglione, September 1999.
406. OCRWM B0000000-01717-5705-00077 Rev 00, *Summary Record of SNF isotopic Comparisons for the Disposal Criticality Analysis Methodology*, M. Nichol, September 1997.
407. OCRWM B0000000-01717-5705-00096 Rev 01, *Summary Report of Commercial Reactor Critical Analyses Performed for Quad Cities Unit 2*, D. P. Henderson, September 1999.
408. OCRWM B0000000-01717-5705-00099 Rev 00, *Selection of MCNP Cross Section Libraries*, K. D. Wright, June 1998.
409. OCRWM B0000000-01717-5705-00101 Rev 0, *Range of Neutronic Parameters For Repository Calculation Analyses*, W. J. Anderson, August 1998.
410. OCRWM B0000000-01717-5705-00138 Rev 00, *Summary Report of Commercial Reactor Critical Analyses Performed for LaSalle Unit 1*, D. P. Henderson, September 1999.
411. OCRWM BBA000000-01717-0200-00009 Rev 00B, *MCNP Evaluation of Laboratory Critical Experiments: Lattice Criticals*, W. E. Wallin, September 1997.
412. OCRWM BBA000000-01717-0200-00045 Rev 00B, *MCNP Evaluation of Laboratory Critical Experiments: Homogeneous Mixture Criticals*, W. J. Anderson, September 1997.
413. OCRWM BBA000000-01717-0200-00046 Rev 00, *CRC Statepoint Reactivity Calculations for Cycles 1A, 1B, 2, 3 and 4 of Crystal River Unit 3*, K. D. Wright, September 1997.

414. OCRWM BBA000000-01717-0200-00047 Rev 00B, *CRC Statepoint Reactivity Calculations for Cycles 5, 6, and 7 of Crystal River Unit 3*, K. D. Wright, September 1997.
415. OCRWM BBA000000-01717-0200-00048 Rev 00, *CRC Statepoint Reactivity Calculations for Cycles 8 and 9 of Crystal River Unit 3*, K. D. Wright, September 1997.
416. TDR-EBS-NU-000003-0, *Summary of the Supplemental Model Reports Supporting the Disposal Criticality Analysis Methodology Topical Report*, D. A. Brownson, September 2004.
417. TDR-EBS-NU-000004-4, *Preclosure Criticality Analysis Process Report*, A. E. Danise, October 2004.
418. TDR-UDC-NU-000001 Rev 02, *Summary Report of Commercial Reactor Criticality Data for Crystal River Unit 3*, M. K. Punatar, June 2001.
419. TDR-UDC-NU-000002 Rev 00B, *Summary Report of Commercial Reactor Criticality Data for Grand Gulf Unit 1*, M. K. Punatar, August 2001.
420. TDR-UDC-NU-000004 Rev 01A, *Summary Report of Commercial Reactor Criticality Data for Three Mile Island Unit 1*, L. Wimmer, July 2001.
421. TDR-UDC-NU-000005 Rev 00, *Summary Report of Code to Code Comparisons Performed for the Disposal Criticality Analysis Methodology*, C. Mays, May 2002.
422. TDR-UDC-NU-000006 Rev 00, *Summary Report of Two-Dimensional Analysis of Radiochemical Assay Samples*, C. Connell, J. M. Scaglione, December 2001.
423. TWP-EBS-MD-000014 Rev 01, *Technical Work Plan for: Risk and Criticality Department*, D. A. Brownson, September 2003.
424. WSMS-CRT-98-0001, *Criticality Evaluation of DOE SNF Codisposal Canister with Melt and Dilute MTR Fuel*, S. M. Revolinski, February 1998.
425. WSRC-TR-97-00345, *Alternative Aluminium Spent Nuclear Fuel Treatment Technology Development Status Report*, R. L. Sindelar et. al., October 1997.
426. YMP/TR-004Q Rev. 02, *Disposal Criticality Analysis Methodology Topical Report*, J. D. Ziegler, November 2003.
427. J. D. Ziegler, DOE-brev till NRC, *Comment on Draft Regulatory Guide DG-3023, Nuclear Criticality Safety Standards for Fuels and Material Facilities*, Proposed revision 1 of Regulatory Guide 3.71. June 2005. Mätningar eller verifiering av utbränning?

ICNC

ICNC 1987 (ISCS '87) (Tokyo)

428. ICNC 1987 (ISCS '87), p. 364, A. H. Wells, *Burnup Credit in Nuclear Criticality Safety Analysis: Application to Spent Fuel Storage/Transport Casks*, October 1987.
429. ICNC 1987, p. 368, H. Toffer, A. H. Wells, *Spent Fuel Critical Masses and Supportive Measurements*, October 1987.
430. ICNC 1987, H. Toffer, A. H. Wells, *Spent Fuel Critical Masses and Supportive Measurements*, October 1987. Presentation material in WHC-SA-0122.
431. ICNC 1987, p. 372, S. Shimura et. al., *Effect of Fission Products on Criticality Safety in LWR Fuel Reprocessing*, October 1987.
432. ICNC 1987, p. 379, L. Maubert, J. C. Puit, *The Burn-Up Consideration in the Criticality Safety of Irradiated LWR Fuel Cycle Plants*, October 1987.

ICNC 1991 (Oxford)

433. ICNC 1991, p. II-58, J. Anno, J. Krebs, *Estimation des Marges de Sécurité dues à 6 P. F. dans les Considerables Irradiés en Transport et Stockage Sous Eau*, September 1991.
434. ICNC 1991, p. II-68, M. C. Brady, T. L. Sanders, *A Validated Methodology for Evaluating Burnup Credit in Spent Fuel Casks*, September 1991.
435. ICNC 1991, p. II-83, N. R. Smith, M. H. Watmough, *Progress in the Development of a Validated Method for Assessing the Reactivity Effects Associated with Fuel Burnup*, September 1991.
436. ICNC 1991, p. II-91, T. Onishi, *Development of Burnup Monitor of LWR Fuel Assemblies for Reprocessing Plants*, September 1991.
437. ICNC 1991, p. II-99, G. Ingram, J. Marchhall, *The Application of DIMPLE Irradiated Fuel Measurements to Burn-Up Credit Validation*, September 1991.
438. ICNC 1991, p. II-107, M. Takano, F. Masukawa, *Effect of Fuel Burnup Distribution on Criticality Safety*, September 1991.
439. ICNC 1991, p. II-113, S. Bierman, *Spent Reactor Fuel Benchmark Composition Data for Code Validation*, September 1991.
440. ICNC 1991, p. II-122, V. Vnukov, B. Rjazanov, *Burnup Credit for Nuclear Safety of Spent Fuel Storage*, September 1991.
441. ICNC 1991, p. II-126, J. M. Conde, M. Recio, F. J. Ramon, *Criticality Safety Evaluation of Fuel Storage Pools in Spain*, September 1991.

ICNC 1995 (Albuquerque)

442. ICNC 1995. Vol I, p. 1B-3, R. L. Bowden, P. R. Thorne, *The Methodology Adopted by British Nuclear Fuels PLC in Claiming Credit for Reactor Fuel Burnup in Criticality Safety Assessments*, September 1995.
443. ICNC 1995. Vol I, p. 1B-11, T. Suzuki et. al., *Exponential Experiments of PWR Spent Fuel Assemblies for Acquiring Subcriticality Benchmarks Usable in Burnup Credit Evaluations*, September 1995.
444. ICNC 1995. Vol I, p. 1B-19, A. Santamarina et. al., *Experimental Validation of Burnup Credit Calculations by Reactivity Worth Measurements in the MINERVE Reactor*, September 1995.
445. ICNC 1995, Vol I, p. 1B-26, M. D. DeHart, C. V. Parks, *Issues Related to Criticality Safety Analysis for Burnup Credit Applications*, September 1995.
446. ICNC 1995, p. 2.11, M. Kurosawa et. al., *Isotopic Composition of Spent Fuels for Criticality Safety Evaluation and Isotopic Composition Database (SFCOMPO)*, September 1995.
447. ICNC 1995, p. 5.3, G. A. Harms, F. J. Davis, J. T. Ford, *The Spent Fuel Safety Experiment*, September 1995.
448. ICNC 1995, p. 5.10, J. Anno et. al., *Description and Exploitation of Benchmarks Involving ¹⁴⁹Sm, A Fission Product Taking Part in the Burnup Credit in Spent Fuels*, September 1995.
449. ICNC 1995, p. 5.18, S. Mitake, O. Sato, N. Yoshizawa, *An Analysis of PWR Fuel Post-Irradiation Examination Data for the Burnup Credit Study*, September 1995.
450. ICNC 1995, Vol I, p. 5.26, M. D. DeHart et al., *Validation of a Method for Prediction of Isotopic Concentrations in Burnup Credit Applications*, September 1995.

451. ICNC 1995, p. 6.44, O. Craig Brown, *Taking Credit for Burnup in Criticality Safety for BWR Fuel – A Fuel Management Perspective*, September 1995.
452. ICNC 1995, p. 11.64, R. I. Ewing, *Burnup Verification Measurements at U.S. Nuclear Utilities Using the FORK System*, September 1995.
453. ICNC 1995, p. 13.34, D. Ito et al., *Simulation of the Subcriticality Determination Methods for Spent Fuel Cask Systems Based on the Cf-252 Source-Driven Neutron Noise Analysis and the Neutron Source Multiplication Method*, September 1995.
454. ICNC 1995, p. 13.51, I. Mitsuhashi et al., *Subcriticality Inference Methods in Every Step of Spent Fuel Loading*, September 1995.
455. ICNC 1995 Supplement, N.T. Gulliford et. al., *The Experimental Validation of Calculational Methods and Data for Burnup Credit*, September 1995.

ICNC 1999 (Versailles)

456. ICNC 1999, pp. , B. Roque, A. Santamarina, N. Thiollay, *Burnup Credit in LWR-MOX Assemblies*, September 1999.
457. ICNC 1999, pp. , L. Markova, *VVER-Related Burnup Credit Calculations*, September 1999.
458. ICNC 1999, p. 576, H-P. Dyck et. al., *Overview on the Burnup Credit Activities at the IAEA*, September 1999.
459. ICNC 1999, pp. , D. A. Thomas et. al., *Disposal Criticality Methodology Using Principal Burnup Credit*, September 1999.
460. ICNC 1999, pp. , R. A. Yearsley, D. N. Simister, T. W. Hicks, *A Review of Criticality Safety Assessments for Radioactive Waste Storage and Disposal Facilities*, September 1999.
461. ICNC 1999, pp. , P. R. Thorne, G. J. O'Connor, R. L. Bowden, *The Current Status of the Methods Adopted by BNFL in Claiming Credit for Burnup in Criticality Safety Assessments*, September 1999.
462. ICNC 1999, pp. , G. Hegyi, G. Hordosy, Csaba Maraczy, *Influence of the Axial Burnup Distribution on Burnup Credit Applications in VVER Systems*, September 1999.
463. ICNC 1999, pp. , Y. Ando et. al., *Burnup Credit Application to Transportation and Storage of Spent BWR Fuel Assemblies (1) Axial Zoning Model*, September 1999.
464. ICNC 1999, pp. , D. Mennerdahl, *Harmonisation of Methods for Criticality Safety Assessment*, September 1999.
465. ICNC 1999, pp. , D. J. Koh, N. Z. Cho, *Studies on Burnup Credit Criticality Safety for Storage and Transportation of PWR Spent Fuel in Korea*, September 1999.
466. ICNC 1999, pp. , A. Tanskanen, *Validation of the KENO-VI and MCNP4B Codes for VVER Criticality Calculations*, September 1999.
467. ICNC 1999, pp. 612-621, N. Thiollay et. al., *Burnup Credit for Fission Product Nuclides in PWR (UO₂) Spent Fuels*, September 1999.
468. ICNC 1999, pp. , A. Terada, D. Ito, T. Kitao, *Burnup Credit Application to Transportation and Storage of Spent Fuel Assemblies (2) – Methods and Results of Calculation*, September 1999.
469. ICNC 1999, pp. 612-621, H. Okuno et. al., *International Studies on Burnup Credit Criticality Safety by and OECD/NEA Working Group (2) Calculation Benchmarks for BWR Spent Fuels*, September 1999.

470. ICNC 1999, pp. 1684-1692, M. Maillot et. al., *Search for an Envelope Axial Burnup Profile for Use in the PWR Criticality Studies with Burnup Credit*, September 1999.
471. ICNC 1999, pp. , T. Suzaki, K. Suyama, T. Kaneko, *Measurement of Criticality Properties of a BWR Spent Fuel Assembly*, September 1999.
472. ICNC 1999, pp., E. Tinkova, *HELIOS System Used in Subcriticality Studies of Hexagonal Lattice*, September 1999.
473. ICNC 1999, pp. , R. R. Paternoster, D. A. Rutherford, *Addressing Critical Experiment Needs for DNFSB Recommendations 97.2*, September 1999.
474. ICNC 1999, pp. , M. Bagarry et. al., *Overview and Consequences of the Changes in the Regulatory Requirements Concerning Criticality Safety in the Transport of Fissile Material*, September 1999.
475. ICNC 1999, pp. , C. Lavarenne, *Taking Burnup Credit Into Account in Criticality Studies: The Situation as It Is Now and the Prospect for the Future*, September 1999.
476. ICNC 1999, pp. , U. Hesse et. al., *KENOREST - A New Criticality and Inventory Calculation System Based on KENO and OREST*, September 1999.
477. ICNC 1999, pp. , M. D. DeHart, B. L. Broadhead, *Investigation of Burnup Credit Issues in BWR Fuel*, September 1999.
478. ICNC 1999, pp. , Y. Pechera, Y. Kovbasenko, *Assurance of Nuclear Criticality Safety in the Design of Spent Fuel Storage Facility of Zaporizhzhya NPP*, September 1999.
479. ICNC 1999, Vol II, p. 603, J. Basselier et al., *Critical Experiment with Spent Fuel for Burn-up Credit Validation (REBUS International Programme)*, September 1999.
480. ICNC 1999, Vol IV. p. 624, J. C. Neuber, *Criticality Analysis of BWR Spent Fuel Storage Facilities Inside Nuclear Power Plants*, September 1999.
481. ICNC 1999, pp. , N. Harris, J. Edge, F. Ogilvie, *The Development of a Burn-Up Credit Criticality Safety Case for the Thorp Reprocessing Plant*, September 1999.
482. ICNC 1999, pp. , K. Oeda et. al., *Criticality Control in the Rokkasho Reprocessing Plant by a Burnup Monitor*, September 1999.
483. ICNC 1999, pp. , A. Smaizys, P. Paskas, *Criticality Analysis of the CASTOR RBMK-1500 and CONSTOR RBMK-1500 Casks Loaded with Spent Nuclear Fuel*, September 1999.
484. ICNC 1999, pp. , V. I. Koulikov et. al., *Nuclear Criticality Safety of Casks with Increased Load of VVER-1000 and VVER-440 Spent Fuel*, September 1999.
485. ICNC 1999, pp. , K. Yoshioka et. al., *Measurement of Burnup and Neutron Emission Rates from a Spent MOX Fuel*, September 1999.
486. ICNC 1999, pp. , Y. Nakahara et. al., *Experimental Verification of Availability of ($^{134}\text{Cs}/^{137}\text{Cs}$)/($^{106}\text{Ru}/^{137}\text{Cs}$) Gamma-Ray Intensity Ratio as Burn-Up Monitor for LWR Fuels*, September 1999.
487. ICNC 1999, pp. , F. Rahnama et. al., *Burnup Simulations for MURR Fuel Assemblies Used in Subcritical Measurements*, September 1999.

ICNC 2003 (Tokaimura)

488. ICNC 2003, pp. 17-22, W. J. Danker, *Current Status of IAEA Activities in Spent Fuel Management*, October 2003.
489. ICNC 2003, pp. 187-191, H. Shin et. al., *Exponential Experiments on PWR Spent Fuel Assemblies*, October 2003.

490. ICNC 2003, pp. 192-196, T. Natsume et. al., *Subcriticality Measurements in PWR Spent Fuel Pit*, October 2003.
491. ICNC 2003, pp. 278-282, R. Blomquist, *OECD/NEA Source Convergence Benchmark program: Overview and Summary of Results*, October 2003.
492. ICNC 2003, pp. 389-394, B. Gmal et. al., *Current Activities in Criticality Analyses of Final Disposal of Spent Nuclear Fuel and Fissile Material Containing Waste in Germany*, October 2003.
493. ICNC 2003, pp. 477-481, J-M Gomit, *CRISTAL-VI: Criticality package for Burn up Credit Calculations*, October 2003.
494. ICNC 2003, pp. 488-493, H. Mochizuki, K. Suyama, H. Okuno, *SWAT2: The Improved SWAT Code System by Incorporating the Continuous Energy Monte Carlo Code MVP*, October 2003.
495. ICNC 2003, pp. 494-499, Y. Ando et. al., *Development and Verification of Monte Carlo Burnup Calculation System*, October 2003.
496. ICNC 2003, pp. 627-632, J. Raby et. al., *Current Studies Related to the Use of Burnup Credit in France*, October 2003.
497. ICNC 2003, pp. 633-638, D. N. Simister, P. D. Clemson, *UK Regulatory Perspective on the Application of Burn-up Credit to the BNFL Thorp Head End Plant*, October 2003.
498. ICNC 2003, pp. 639-644, G. Caplin et. al., *Specific Application of Burnup Credit for MOX PWR Fuels in the Rotary Dissolver*, October 2003.
499. ICNC 2003, pp. 645-649, P. Baeten et. al., *The REBUS Experimental Programme for Burn-up Credit*, October 2003.
500. ICNC 2003, pp. 650-655, L. Markova, *Sensitivity Study Applied to the CB4 VVER-440 Benchmark on Burnup Credit*, October 2003.
501. ICNC 2003, pp. 656-660, G. Hordosy, *Source Convergence Problems in the Application of Burn-up Credit for WWER-440 Fuel*, October 2003.
502. ICNC 2003, pp. 661-665, Y. Kovbasenko, Y. Bilodid, M. Yeremenko, *Comparative Analysis of Isotope Composition of VVER-1000 Spent Fuel Depending on Their Manufactory and Operation Conditions*, October 2003.
503. ICNC 2003, pp. 666-671, J. Anno et. al., *French Fission Products Experiments performed in Cadarache and Valduc. Results Comparison*, October 2003.
504. ICNC 2003, pp. 672-677, J-C Neuber, *Generation of Bounding Axial Burnup Profiles as a Continuous Function of Average Burnup*, October 2003.
505. ICNC 2003, pp. 678-683, S. H. Lee, J. G. Ahn, H. R. Hwang, *Conceptual Cask with Burnup Credit*, October 2003.
506. ICNC 2003, pp. 684-689, J .C. Wagner, *Evaluation of Burnup Credit for Accommodating PWR Spent Nuclear Fuel in High-Capacity Cask Designs*, October 2003.
507. ICNC 2003, pp. 690-693, Y. Nomura, *Investigation of Several Methods to Set Burnup for Criticality Safety Assessment of Spent Fuel Transport Casks*, October 2003.
508. ICNC 2003, pp. 694-699, A. Barreau et. al., *Parametric Studies of the Effects of MOx Environment and Control Rods for PWR-UOx Burnup Credit Implementation*, October 2003.
509. ICNC 2003, pp. 700-705, Y. Bilodid, M. Yeremenko, Y. Kovbasenko, *Testing the SAS2H SCALE Control Module on VVER Type Fuel*, October 2003.
510. ICNC 2003, pp. 706-710, J. Gulliford et. al., *The Implementation of Burnup Credit Based Criticality Safety Assessment in the THORP Head End Plant*, October 2003.

511. ICNC 2003, pp. 711-716, T. Kikuchi et. al., *Application of Gadolinia Credit to Cask Transportation of BWR-STEP3 SFAs*, October 2003.
512. ICNC 2003, pp. 717-720, H. Toubon, *Burn-up Credit Applications for UO₂ and MOX Fuel Assemblies in AREVA/COGEMA*, October 2003.
513. ICNC 2003, pp. 721-728, M. Yeremenko, Y. Kovbasenko, Y. Bilodid, *Preparation and Testing ORIGEN-ARP Library for VVER Fuel Design*, October 2003.
514. ICNC 2003, pp. 865-870, B. Roque et. al., *The French Post Irradiation Examination Database for the Validation of Depletion Calculation Tools*, October 2003.
515. ICNC 2003, pp. 871-876, N. Shinohara et. al., *Recent Activities on the Post-Irradiation Analyses of Nuclear Fuels and Actinide Samples at JAERI*, October 2003.
516. ICNC 2003, pp. 877-883, S. Koyama, T. Namekawa, K. Tsujimoto, *Current Status of PIE Activities in O-arai Engineering Center of JNC on FBR MOX Fuel*, October 2003.
517. ICNC 2003, pp. 884-889, I. Günther-Leopold, B. Wernli, Z. Kopjtic, *Characterization of Spent Nuclear Fuel by an Online Combination of Chromatographic and Mass Spectrometric Techniques*, October 2003.
518. ICNC 2003, pp. 890-896, K. Suyama et. al., *Improvements to SFCOMPO – A Database on Isotopic Composition of Spent Nuclear Fuel*, October 2003.

ANS Topicals on Nuclear Criticality Safety

ANS/NCSD Topical Meeting 1985 (Jackson, Wyoming)

519. NCSD/1985, p. 19, R. L. Eng, *Criticality Implications of Extended Fuel Burnup*, September 1985. Only abstract (document was to be distributed at the meeting)
520. NCSD/1985, p. 135, J. R. McCowan, *Generic Reactivity Equivalence of PWR Fuel in Spent Fuel Storage Racks*, September 1985.
521. NCSD/1985, p. 254, J. R. McCowan, *Criticality Analyses for TMI-2 Defueling*, September 1985.

ANS/NCSD Topical Meeting 1989 (San Francisco)

522. NCSD 1989, p. 304, W. A. Boyd et. al., *IFBA Credit in the Shearon Harris Fuel Racks with Vantage 5 Fuel*, September 1989.
523. NCSD 1989, p. 326, J. K. Boshoven, S-D Su, *Criticality Safety Margins in a Spent Fuel Transportation Cask with Burnup Credit*, September 1989.

ANS/NCSD Topical Meeting 1993 (Nashville)

524. NCSD 1993, p. 139, S. A. Rice, P. R. Dye, P. R. Thorne, *A Physics Based Overview of the Operational Criticality Plant Safety Case for the THORP Fuel Dissolver*, September 1993.

ANS/NCSD Topical Meeting 1997 (Chelan)

525. NCSD 1997, p. 271, J. Anno et. al., *Planned Experimental Program Qualifying the Safety Margins Given by 6 Selected Fission Products in Spent Fuels*, September 1997.
526. NCSD 1997, p. 279, A. Santamarina et. al., *The French Experimental Programme on Burnup Credit*, September 1997.
527. NCSD 1997, p. 286, E. Fuentes, D. Lancaster, M. Rahimi, *Isotopic and Criticality Validation for Actinide-Only Burnup Credit*, September 1997.
528. NCSD 1997, p. 293, D. A. Thomas et. al., *The Use of Spent Fuel Isotopics in the Disposal Criticality Analysis Methodology*, September 1997.
529. NCSD 1997, p. , J. R. Chandler, E. F. Trumble, *Use of Bias, Uncertainty and Subcritical Margin at the Savannah River Site*, September 1997. Published separately as WSRC-MS-96-0682.

ANS/NCSD Topical Meeting 2001 (Reno)

530. NCSD 2001, D. N. Simister, P. D. Clemson, *UK Regulatory Perspective on the Application of Burn-Up Credit in Plant Criticality Safety Cases*, November 2001.
531. NCSD 2001, J. Anno et al., *Status of the Joint French IPSN/COGEMA Qualification Programme of Fission Products*, November 2001.
532. NCSD 2001, S. K. Kessler et al., *Use of Burnup Credit for Criticality Safety for the Hanford Spent Nuclear Fuel Project*, November 2001.
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- 545. NCS D 2001, A. Lebrun et al., *Average Burnup and Axial Burnup Profile Measurement for Burnup Credit Application*, November 2001.
- 546. NCS D 2001, C. Lavarenne et al., *A New Method to Take Burnup Into Account in Criticality Studies Considering an Axial Profile of Burn-Up Plus some Fission Products*, November 2001.
- 547. NCS D 2001, C. E. Sanders, J. C. Wagner, *Impact of Integral Burnable Absorbers on PWR Burnup Credit Criticality Safety Analyses*, November 2001.
- 548. NCS D 2001, C. E. Sanders, M. D. DeHart, *Comparison Computational Estimations of Reactivity Margin From Fission Products and Minor Actinides in PWR Burnup Credit*, November 2001.

ANS/NCS D Topical Meeting 2005 (Knoxville)

- 549. NCS D 2005, P. D'hondt et. al., *The REBUS-PWR Experimental Programme for Burn-Up Credit*, September 2005.
- 550. NCS D 2005, D. Lancaster, A. Zimmer, J. Razvi, *Burnup Credit for the GA-4 Cask Using 2005 Methodologies*, September 2005.
- 551. NCS D 2005, A. H. Wells, D. A. Thomas, *Methods for Validation of Burnup Credit Calculations*, September 2005.
- 552. NCS D 2005, D. E. Mueller, B. T. Rearden, *Sensitivity Coefficient Generation for a Burnup Credit Cask Model Using TSUNAMI-3D*, September 2005.
- 553. NCS D 2005, R. Kilger, B. Gmal, *Burnup calculations with KENOEST 03T01 and associated criticality studies for spent fuel samples from Takahama-3 reactor*, September 2005.
- 554. NCS D 2005, J. C. Wagner, D. E. Mueller, *Updated Evaluation of Burnup Credit for Accommodating PWR Spent Nuclear Fuel in High-Capacity Cask Designs*, September 2005.

ANS Transactions

- 555. TANS-WM-1972, **15**, 370, H. Toffer, R. L. Miller, *Criticality Control of Parameters for Discharged Fuel*, 1972.
- 556. TANS-WM-1975, **22**, 477, H. Toffer, L. A. Nielsen, *Subcritical Measurements with Irradiated and Unirradiated Uranium Metal Fuel Elements*, November 1975.
- 557. TANS-AM-1976, **23**, 508, T. Gozani, *An Instrumented Pool – A Possible Way to Increase Storage Capacity*, June 1976.
- 558. TANS-1980, **33**, 657, S. H. Levine, M. A. Schultz, D. Chang, *A k_{∞} Meter for Spent Nuclear Fuel Storage Pools*, 1980.
- 559. TANS-WM-1981, (DPSP-AFR-81-6-3), C. E. Jewell, *Criticality Consideration of Interim Spent Fuel Storage*, November 1981.
- 560. TANS-AM-1984, **46**, 102, C. C. Wang, R. E. Best, *Storage of Extended Burnup and Consolidated Fuel in the REA-2023 Cask*, June 1984.
- 561. TANS-AM-1984, **46**, 457, A. H. Wells, *A Method for Estimating the Value of Burn-Up Credit for Nuclear Criticality Safety of Spent Fuel Shipments*, June 1984.
- 562. TANS-WM-1984, **47**, 094, N. P. Godstein, R. W. Chickering, D. Dzladosz, *A System to Measure the Burnup of Spent Fuel Assemblies*, November 1984.
- 563. TANS-WM-1985, **50**, 300, M. Todoscov, J. F. Carew, *Effects of Assembly Local Power Distribution on Storage Rack Criticality*, November 1985.

564. TANS-WM-1986, **53**, 103, A. L. Snow, J. R. Thornton, *Extended Burnup: Ex-Core and Accident Considerations*, November 1986.
565. TANS-WM-1986, **53**, 104, *Optimization of Spent-Fuel Storage Equipment with Extended Burnup*, November 1986.
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567. TANS-WM-1986, **53**, 108, A. H. Wells, *Effects of High Burnup on Spent-Fuel Casks*, November 1986.
568. TANS-WM-1986, **53**, 108, J. E. Matheson, *The Application of High Burnup Experience to Consolidation Equipment Design*, November 1986.
569. TANS-WM-1987, **55**, 389, A. J. Koudelka, *The "Law of Substitution" for Mixed Arrays*, November 1987.
570. TANS-WM-1987, **55**, 391, J. P. Roberts, *Burnup Credit Consideration in Dry Spent-Fuel Storage Licensing*, November 1987.
571. TANS-WM-1987, **55**, 391, T. L. Sanders, R. M. Westfall, E. L. Wilmer, *Burnup Credit in the Design of Spent-Fuel*, November 1987.
572. TANS-WM-1987, **55**, 393, W. A. Boyd, *Determining Burnup Credit Requirements in Spent-Fuel Storage Racks Using Reactivity Equivalencing*, November 1987.
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574. TANS-WM-1987, **55**, 396, W. A. Boyd, G. N. Wrights, *Past Experience and Future Needs for the Use of Burnup Credit in LWR Fuel Storage*, November 1987.
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576. TANS-WM-1987, **55**, 398, D. G. Napolitano, *Analysis of Burnup Credit in Fuel Storage with CASMO*, November 1987.
577. TANS-WM-1987, **55**, 400, W. L. Brooks, *Credit for Burnup in Spent-Fuel Storage Rack Design – Regulatory Perspective*, November 1987.
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580. TANS-WM-1987, **55**, 402, J. T. Mihalcz, *Evaluation of the ²⁵²Cf-Source Driven Neutron Noise Analysis Method for Measuring the Sub-criticality of LWR Fuel Storage Casks*, November 1987.
581. TANS-WM-1987, **55**, 403, C. O. Brown, *Burnup Credit in the Storage of LWR Fuel – Conceptual Considerations*, November 1987.
582. TANS-WM-1988, **56**, 328, A. H. Wells, *Burnup Credit for Storage and Transportation Casks*, June 1988.
583. TANS-1990, **62**, 305, D. Napolitano, D. Adli, *Validation of the YAEC Burnup Credit Methodology*, November 1990.
584. TANS-1990, **62**, 306, M. H. Watmough, A. M. Evans, *Burnup Credit: Calculational Schemes and Related Considerations*, November 1990.
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586. TANS-1990, **62**, 310, R. B. Kidman, *MCNP Criticality Calculations Using Multigroup and Continuous-Energy Cross Sections*, November 1990.
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588. TANS-1990, **62**, 313, C. R. Marotta, W. H. Lake, T. L. Sanders, *Alternative Certification Options for a Burnup Credit Spent-Fuel Cask*, November 1990.
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590. TANS-1990, **62**, 315, D. Napolitano, D. Adli, *Burnup Credit Analysis of a 31-Assembly Cask*, November 1990.
591. TANS-1990, **62**, 317, M. C. Brady, C. V. Parks, C. R. Marotta, *End Effects in the Criticality Analysis of Burnup Credit Casks*, November 1990.
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593. TANS-1990, **62**, 319, B. L. Broadhead, *Burnup Feasibility for BWR Spent-Fuel Shipments*, November 1990.
594. TANS-1990, **62**, 322, J. T. Mihalcz, B. L. Broadhead, *Feasibility of Spent LWR Fuel Subcritical Measurements by the Californium Noise Method*, November 1990.
595. TANS-1990, **62**, 324, G. D. Spriggs, *Subcritical Measurements of Spent-Fuel Shipping Casks*, November 1990.
596. TANS-1990, **62**, 325, A. H. Wells, *A Benchmarked Multipurpose Burnup Meter*, November 1990.
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598. TANS-1990, **62**, 327, H. Forsström, L. Agrenius, S. Helmersson, *Burnup Credit in the Central Storage Facility for Spent Fuel in Sweden*, November 1990.
599. TANS-1990, **62**, 328, W. A. Boyd, M. W. Fecteau, *Effect of Axial Burnup on Fuel Storage Rack Burnup Credit Reactivity*, November 1990.
600. TANS-1991, **64**, 334, G. E. Whitesides, *Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors: ANS-8.17-1984*, June 1991.
601. TANS-1993, **68**, 241, S. M. Bowman, H. Taniuchi, *Burnup Credit Validation of SCALE-4 Using Mixed-Oxide Critical Experiments*, June 1993.
602. TANS-1993, **68**, 242, P. R. Thorne, S. A. Rice, *Burnup in Operations in the British Nuclear Fuel Industry*, June 1993.
603. TANS-1993, **68**, 243, S. M. Bowman, O. W. Hermann, M. C. Brady, *Burnup Credit Validation of SCALE-4 Using Light-Water Reactor Criticals*, June 1993.
604. TANS-1994, **71**, 275, C. T. Rombough, S. H. Martonak, J. Walkin, *Using ORIGEN/KENO to Calculate Burnup Credit for Spent-Fuel Pool Criticality Analyses*, November 1994.
605. TANS-1996, **75**, 180, M. Rahimi, D. B. Lancaster, *Isotopic Validation for PWR Actinide-Only Burnup Credit Using Mihama-3 Data*, November 1996.
606. TANS-1996, **75**, 201, G. E. Whitesides, *Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors: ANS-8.17-1984(R89)*, November 1996.
607. TANS-1997, **76**, 52, W. Lake, *Cost-Savings Potential for Transport and Storage of Spent Nuclear Fuel Available from Burnup Credit*, June 1997.
608. TANS-1997, **76**, 53, M. Rahimi, *Isotopic Biases for Actinide-Only Burnup Credit*, June 1997.
609. TANS-1997, **76**, 53, M. D. DeHart, C. V. Parks, *SCALE Depletion Calculations Based on Two-Dimensional PWR Assembly Flux Solutions*, June 1997.
610. TANS-1997, **76**, 57, E. Fuentes, D. B. Lancaster, *Criticality Validation for Burnup Credit Using Recycle Plutonium Criticals*, June 1997.

611. TANS-1997, **76**, 59, C. H. Kang, D. B. Lancaster, *End-Effect k_{eff} Bias Curve for Actinide-Only Burnup Credit Casks*, June 1997.
612. TANS-1997, **76**, 60, D. B. Lancaster, E. Fuentes, *Appropriate Burnup Measurements for Transportation Burnup Credit*, June 1997.
613. TANS-1997, **76**, 61, W. Doering, D. A. Thomas, *Disposal Criticality Analysis Methodology*, June 1997.
614. TANS-1997, **76**, 63, D. A. Thomas, W. Doering, *Use of Spent-Fuel Isotopics in the Disposal Criticality Analysis Methodology*, June 1997.
615. TANS-1997, **77**, 223, D. Bowen, R. D. Busch, *Using ORIGEN and MCNP to Calculate Reactor Criticals and Burnup Effects*, November 1997.
616. TANS-1997, **77**, 224, D. Lancaster, E. Fuentes, C. Kang, *Value of Burnup Credit Beyond Actinides*, November 1997.
617. TANS-1998, **79**, 276, D. Lancaster, C. Kang, W. Lake, *Value of ^{236}U to Actinide-Only Burnup Credit*, November 1998.
618. TANS-1998, **79**, 277, R. J. Cacciapouti et. al., *Determination of the Uncertainty in Assembly Average Burnup*, November 1998.
619. TANS-1998, **79**, 278, G. I. Maldonado, E. Fuentes, *Determination of Limiting Cask Configuration by Simulated Annealing*, November 1998.
620. TANS-1998, **79**, 279, P. J. Finck, C. G. Stenberg, *Fission Product Margin in Burnup Credit*, November 1998.
621. TANS-1998, **79**, 281, M. J. Anderson, D. P. Henderson, *BWR Benchmarking for Yucca Mountain Disposal Criticality Methodology*, November 1998.
622. TANS-1998, **79**, 282, C. E. Olson et. al., *The Fork+ Burnup Measurement System; Design and First Measurement Campaign*, November 1998.
623. TANS-1999, **80**, M. Maillot, E. Guillou, D. Biron, *Search for an Envelope Axial Burnup Profile for Use in PWR Criticality Studies with Burnup Credit*, June 1999.
624. TANS-1999, **81**, D. P. Henderson, D. A. Salmon, *Disposal Criticality Analysis Methodology: BWR Benchmarks*, November 1999.
625. TANS-2000, **82**, M. Zucchetti, M. R. Gual, *A Comparison of Burnup Credit Calculations in the VVER-440 Juragua-1 and PWR-PUN Nuclear Reactors*, June 2000.
626. TANS-2000, **82**, M. R. Gual, *Recalculations of Criticality of the VVER Burnup Credit Benchmark No. 1*, June 2000.
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631. TANS-2000, **83**, 90, C. M. Hopper, *Sensitivity and Uncertainty Analysis Methods for Establishing Area of Applicability and Subcritical Margins*, November 2000.
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634. TANS-2000, **83**, 98, B. T. Rearden, R. L. Childs, *Prototypical Sensitivity and Uncertainty Analysis Codes for Criticality Safety with the SCALE Code System*, November 2000.
635. TANS-2000, **83**, 100, B. L. Broadhead, R. L. Childs, C. M. Hopper, *Illustrative Examples of Least-Squares Methods for Criticality Safety*, November 2000.
636. TANS-2000, **83**, 103, B. T. Rearden et. al., *Prototypic Applications of Sensitivity and Uncertainty Analysis for Experimental Needs*, November 2000.
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638. TANS-2000, **83**, 115, D. E. Carlson, C. J. Withee, *Regulatory Status of Burnup Credit for Spent-Fuel Storage and Transport Casks*, November 2000.
639. TANS-2000, **83**, 115, T. W. Doering, *Dry Cask Storage and Transportation Burnup Credit*, November 2000.
640. TANS-2000, **83**, 116, W. H. Lake, H. P. Dyck, *IAEA TCM 2000: An International Meeting on Burnup Credit Applications*, November 2000.
641. TANS-2000, **83**, 117, D. D. Ebert, R. Y. Lee, *NRC Burnup Credit Research Program*, November 2000.
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644. TANS-2000, **83**, 121, M D. DeHart, *A Statistical Method for Estimating the Net Uncertainty in the Prediction of k Based on Isotopic Uncertainties*, November 2000.
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646. TANS-2000, **83**, 124, C. Connell, R. Kochendarfer, *The Effect of Half-Life and Branching Fraction Uncertainties on the Effective Neutron Multiplication Factor*, November 2000.
647. TANS-2000, **83**, 124, B. L. Broadhead, *Value rankings of Selected Critical Experiments for Burnup Credit Validations*, November 2000.
648. TANS-2000, **83**, 126, D. B. Lancaster, C. T. Rombough, H. Spilker, *Actinide-Only Burnup Credit License Application: The CASTOR X/32 S Cask Modeling Details*, November 2000.
649. TANS-2000, **83**, 128, P. M. Leary, M. L. Pitts, *Effects of Integral Burnable Absorbers on PWR Spent Nuclear Fuel*, November 2000.
650. TANS-2000, **83**, 130, J. C. Wagner, C. V. Parks, *Impact of Burnable Poison Rods on PWR Burnup Credit Criticality Safety Analyses*, November 2000.
651. TANS-2000, **83**, 135, D. P. Henderson, *PWR Radichemical Assay Benchmarks Using SAS2H and CASMO*. November 2000.
652. TANS-2000, **83**, 136, M. L. Pitts, P. M. Leary, *Modeling BWR Spent-Fuel Isotopics with SAS2H and CASMO-3*, November 2000.
653. TANS-2000, **83**, 137, J. J. Sapyta, C. W. Mays, J. W. Pegram, *Use of Reactor-Follow Data To Determine Biases and Uncertainties for PWR Spent Nuclear Fuel*, November 2000.
654. TANS-2000, **83**, 138, J. M. Scaglione et. al., *Applicability of CRC Benchmark Experiments for Burnup Credit Validation*, November 2000.
655. TANS-2000, **83**, 140, W. J. Anderson, P. M. O'Leary, J. M. Scaglione, *Selection of Reactor Criticals as Benchmarks for Spent Nuclear Fuels*, November 2000.

656. TANS-2000, **83**, 141, W. J. Anderson, *Temperature Effects on Reactivity of Commercial Reactor Criticals as Benchmarks for Spent Nuclear Fuels*, November 2000.
657. TANS-2000, **83**, 153, B. O. Kidd, *Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors: ANS-8.17-1984(R97)*, November 2000.
658. TANS-2000, **83**, 179, C. T. Rombough, D. B. Lancaster, H. Spilker, *Criticality Characteristics of the CASTOR X/32 Storage and transportation Cask*, November 2000.
659. TANS-2001, **84**, 288, S. Janski, *Evaluation of Criticality Risks During the Storage of Irradiated Assemblies*, June 2001.
660. TANS-2001, **84**, 292, D. B. Lancaster et. al., *The CASTOR X/32S Method of Covering Misloading Concerns*, June 2001.
661. TANS-2001, **84**, 292, C. T. Rombough et. al., *The CASTOR X/32S Method of Covering Misloading Concerns*, June 2001.
662. TANS-2001, **84**, 352, P. M. O’Leary, J. M. Scaglione, *An Empirical Approach to Bounding the Axial Reactivity Effects of PWR Spent Nuclear Fuel*, June 2001.
663. TANS-2001, **85**, 268, A. Meister, P. Grimm, R. Chawla, *Neutronics Design of LWR-PROTEUS*, November 2001.
664. TANS-2001, **85**, 269, D. Fujiwara, S. Kosaka, *Criticality Analyses of BWR Spent Nuclear Fuel Using CASMO/CITATION*, November 2001.
665. TANS-2001, **85**, 441, D. Dziadosz, P. M. O’Leary, *Spent Nuclear Fuel Burnup Verification Through the Use of Reactor Records*, November 2001.
666. TANS-2002, **86**, 28, M. A. Feltus, *Waste Management Research Projects Sponsored by the DOE Nuclear Energy Research Initiative Program*, June 2002.
667. TANS-2002, **86**, 98, C. E. Sanders, J. C. Wagner, *Investigation of Average and Pin-Wise Burnup Modeling of PWR Fuel*, June 2002.
668. TANS-2002, **86**, 100, C. E. Sanders, M. D. DeHart, *Computational Benchmark of SAS2D Against Spent Fuel Samples From the Takahama-3 Reactor*, June 2002.
669. TANS-2002, **86**, 102, D. B. Lancaster, *PWR Burnup Credit Using Both Belts and Suspenders*, June 2002.
670. TANS-2002, **87**, 105, J. M. Scaglione, *Isotopic Bias and Uncertainty for Burnup Credit Applications*, November 2002.
671. TANS-2002, **87**, 107, G. A. Harms et. al., *Rhodium Experiments in the NERI Burnup Credit Critical Experiment*, November 2002.
672. TANS-2002, **87**, 108, H. Joo, *Application of the RCP01 Code to Depletion of a PWR Spent Nuclear Fuel Sample*, November 2002.
673. TANS-2002, **87**, 110, H. Shin et. al., *Exponential Experiment System Setup and Performance Test*, November 2002.
674. TANS-2002, **87**, 148, Y. naito, J. Yang, *Benchmark Results: Irradiated Pin Cell Array*, November 2002.
675. TANS-2002, **88**, 84, H. R. Radulescu, *SAS2H/Radiochemical Assay Comparisons for Limeric BWR Spent Fuel Samples*, June 2003.
676. TANS-2003, **88**, 181, A. H. Wells, *Burnup Credit Isotopic Validation with Commercial Reactor Criticals*, June 2003.
677. TANS-2003, **88**, 439, S. B. Baker, *Development of the Calvert Cliffs Isotopics Benchmark Specification*, June 2003.
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679. TANS-2003, **88**, 524, D. B. Lancaster, *Effect of Additional Chemical Assay Data on Actinide-Only Burnup Credit*, June 2003.
680. TANS-2003, **89**, 120, J. C. Wagner, *Impact of Soluble Boron Modeling for PWR Burnup Credit Criticality Safety Analyses*, November 2003.
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682. TANS-2003, **89**, 585, S. B. Baker, *TransLAT Benchmark of ANS Benchmark Committee B&W Criticals*, November 2003.
683. TANS-2004, **90**, 109, D. Mennerdahl, *A Declaration of Independence – What Is It Worth?*, June 2004.
684. TANS-2004, **90**, 138, T. M. Lloyd, J. E. Hopf, S. E. Sisley, *An Automated Approach for Dry Fuel Transportation Criticality Qualification*, June 2004.
685. TANS-2004, **90**, 144, D. Eghbali, *Use of Burnup Credit as a Safety Factor in Handling of Spent Fuel at SRS*, June 2004.
686. TANS-2004, **91**, 632, B. O. Kidd, *Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors: ANS-8.17-1984(R97)*, November 2004.
687. TANS-2004, **91**, 637, D. Lancaster, *Development of the Burnup Credit Standard, ANSI/ANS-8.27*, November 2004.
688. TANS-2004, **91**, 667, M. D. DeHart, L. M. Petrie, *Integrated KENO V.a Monte Carlo Transport for Multidimensional Depletion Within SCALE*, November 2004.
689. TANS-2004, **91**, 670, B. D. Murphy, I. C. Gauld, *Spent-Fuel Decay Heat Investigations for BWR Assemblies Using Both One- and Two-Dimensional Model Simulations*, November 2004.
690. TANS-2004, **91**, 673, I. C. Gauld, *Automated Depletion Analysis of PBMR Fuel Using SCALE*, November 2004.
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692. TANS-2004, **91**, 747, S. Yimaz et. al., *Optimized Innovative Burnable Poison Concepts for Advanced PWR Fuel Management*, November 2004.
693. TANS-2005, **92**, 376, T. M. Lloyd, B. D. Thomas, *Software Validation of SCALE Pre- and Post-Processors*, June 2005.
694. TANS-2005, **92**, 584, T. M. Lloyd, J. E. Hopf, *A SCALE/MCNP Method for Spent Fuel Transportation Criticality Qualification*, June 2005.
695. TANS-2005, **92**, 651, J. C. Davis, J. C. Lee, *Comparison of Monte Carlo and Deterministic Codes for LWR Fuel Cycle Analysis*, June 2005.
696. TANS-2005, **92**, 660, H. R. Trellue, G. W. McKinney, J. Durkee, *Burnup Capability Added to MCNPX*, June 2005.
697. TANS-2005, **92**, 674, D. E. Burk et. al., *Isotopic Compositions of Material Used In a Radiological Dispersal Device Determined by Forward Model Calculations*, June 2005.
698. TANS-2005, **92**, 747, S. M. Bowman, *Overview of Advances in SCALE Development*, June 2005.
699. TANS-2005, **92**, 763, M. D. DeHart, *Assessment of TRITON and PARCS for Full-Core MOX Fuel Calculations*, June 2005.
700. TANS-2005, **92**, 767, S. M. Bowman, J. E. Horwedel, *GeeWiz: Integrated User Interface for SCALE*, June 2005.
701. TANS-2005, **93**, 263, D. E. Mueller, G. A. Harms, *Using the SCALE 5 TSUNAMI-3D Sequence in Critical Experiment Design*, November 2005.

702. TANS-2005, **93**, 267, D. G. Bowen, *ENDF/B Cross-Section Evolution for Criticality Safety Burnup Credit Applications*, November 2005.
703. TANS-2005, **93**, 351, H. O. Zabunoglu, E. Metin, Y. O. Ozkok, *Calculation of Uranium-236 Penalty for Recycle Uranium Fuels*, November 2005.

Nuclear Technology

704. NT-1978, **38**, p. 367-373, K. P. Termaat, *Reactivity Behaviour of a Reactor Core Loaded with Gadolinium-Poisoned Fuel Assemblies*, May 1978.
705. NT-1995, **110**, p. 1, B. L. Broadhead, *Feasibility Assessment of Burnup Credit in the Criticality Analysis of Shipping Casks with Boiling Water Reactor Spent Fuel*, April 1995.
706. NT-1995, **110**, p. 22, J. M. Conde, M. Recio, *Evaluation of Burnup Credit for Fuel Storage Analysis – Experience in Spain*, April 1995.
707. NT-1995, **110**, pp. 40., Y. Naito, M. Takano, M. Kurosawa, *Study on the Criticality Safety Evaluation Method for Burnup Credit in Japan*, April 1995.
708. NT-1995, **110**, pp. 53-, S. M. Bowman, M. D. DeHart, C. V. Parks, *Validation of SCALE-4 for Burnup Credit Applications*, April 1995.
709. NT-1998, **122**, pp. 255-264, M. Maucec. M. Ravnik, B. Glumac, *Criticality Analysis of the Multiplying Material Inside the Chernobyl Sarcophagus*, June 1998.
710. NT-1998, **123**, pp. 259-267, K. D. Wright, J. S. Tulenko, E. T. Dugan, *Comparison Between MCNP and Critical Experiments – A Determination of Bias Values to be Utilized in Licensing Calculations for High-Level Radioactive Waste Disposal*, September 1998.
711. NT-1999, **125**, pp. 255-270, D. Lancaster et. al., *Actinide-Only Burnup Credit for Pressurized Water Reactor Spent Nuclear Fuel – I: Methodology Overview*, March 1999.
712. NT-1999, **125**, pp. 271-291, E. Fuentes, D. Lancaster, M. Rahimi, *Actinide-Only Burnup Credit for Pressurized Water Reactor Spent Nuclear Fuel – II: Validation*, March 1999.
713. NT-1999, **125**, pp. 292-304, C. H. Kang, D. B. Lancaster, *Actinide-Only Burnup Credit for Pressurized Water Reactor Spent Nuclear Fuel – III: Bounding Treatment of Spatial Burnup Distributions*, March 1999.
714. NT-1999, **126**, pp. 303-318, J. Ahn, *Criticality Safety Assessment for a Conceptual High-Level Waste Repository in Water-Saturated Geologic Media*, June 1999.
715. NT-1999, **127**, pp. 1-23, L. C. Leal, *Automatic Rapid Process for the Generation of Problem-Dependent SAS2H/ORIGEN-S Cross-Section Libraries*, July 1999.
716. NT-2000, **128**, pp. 285-299, W. S: Charlton et. al., *Comparison of Calculated and Measured ^{237}Np , ^{241}Am , and ^{241}Am as a function of the $^{240}\text{Pu}/^{239}\text{Pu}$ Isotopic Ratio in Spent Fuel*, December 1999.
717. NT-2000, **129**, pp. 141-151, T. Nakamura et. al., *Boiling Water Reactor Fuel Behaviour under Reactivity-Initiated-Accident Conditions at Burnup of 41 to 45 GWd/tonne U*, February 2000.
718. NT-2000, **130**, pp. 9-17, Y. S. Kim, *Fission Gas Release from UO_{2+x} in Detective Fuel Rods*, April 2000.
719. NT-2000, **131**, pp. 385-394, K. Hesketh, M. Delpech, E. Sartori, *The Physics of Plutonium Fuels – A Review of Organization for Economic Cooperation and Development/Nuclear Energy Agency Activities*, September 2000.

720. NT-2000, **132**, pp. 325-338, A. Persic, M. Ravnik, T. Zagar, *TRIGA Mark II Criticality Benchmark Experiment with Burned Fuel*, December 2000.
721. NT-2001, **135**, pp. 131-145, S. Jacobsson et. al., *A Tomographic Method for Verification of the Integrity of Spent Nuclear Fuel Assemblies-I: Simulation Studies*, August 2001.
722. NT-2001, **135**, pp. 146-153, S. Jacobsson et. al., *A Tomographic Method for Verification of the Integrity of Spent Nuclear Fuel Assemblies-II: Experimental Investigation*, August 2001.
723. NT-2001, **135**, pp. 216-229, A. Lebrun, G. Bignan, *Nondestructive Assay of Nuclear Low-Enriched Uranium Spent Fuels for Burnup Credit Application*, September 2001.
724. NT-2001, **135**, pp. 230-240, M. Takahashi, *An Estimation Method for Off-Gas Sources in a Boiling Water Reactor with Nondefective Fuel*, September 2001.
725. NT-2001, **136**, pp. 24-36, W. S. Charlton, W. D. Stanbro, *Monitors for the Prediction of Alternate Nuclear Material Concentrations for Pressurized Water Reactor Spent Fuel*, October 2001.
726. NT-2001, **136**, pp. 99, R. P. Rechard et. al., *Unfavorable Conditions for Nuclear Criticality Following Disposal of Transuranic Waste at the Waste Isolation Pilot Plant*, October 2001.
727. NT-2001, **136**, pp. 130-140, J. C. Wagner, C. V. Parks, *A Critical Review of the Practice of Equating the Reactivity of Spent Fuel to Fresh Fuel in Burnup Credit Safety Analyses for PWR Spent-Fuel Pool Storage*, October 2001.
728. NT-2002, **137**, pp. 111-126, Y. Nakahara et. al., *Nuclide Composition Benchmark Data Set for Verifying Burnup Codes on Spent Light-Water Reactor Fuels*, February 2002.
729. NT-2002, **138**, pp. 1-16, V. V. Konyashov, A. M. Krasnov, *Radioactive Fission Product Release from Defective Light Water Reactor Fuel Elements*, April 2002.
730. NT-2002, **138**, pp. 97-110, K. Suyama, H. Mochizuki, *Revised Burnup Code System SWAT: Description and Validation Using Postirradiation Data*, May 2002.
731. NT-2002, **139**, pp. 91-126, J. C. Wagner, C. E. Sanders, *Investigation of the Effect of Fixed Absorbers on the Reactivity of PWR Spent Nuclear Fuel for Burnup Credit*, August 2002.
732. NT-2002, **140**, pp. 255-265, H. Okuno, T. Sakai, *Burnup Importance Function Introduced to Give an Insight Into the End Effect*, December 2002.
733. NT-2003, **141**, pp. 211-219, M. Logar, R. Jeraj, B. Glumac, *The Effect of Pitch, Burnup, and Absorbers on a TRIGA Spent-Fuel Pool Criticality Safety*, February 2003.
734. NT-2003, **143**, pp. 358-363, A. C. Fernandes, *Monte Carlo Modeling of the Portuguese Research Reactor Core and Comparison with Experiental Measurements*, September 2003.
735. NT-2003, **144**, pp. 186-200, R. E. Einziger et. al., *Examination of Spent Pressurized Water Reactor Fuel Rods After 15 Years in Dry Storage*, Nov. 2003.
736. NT-2005, **149**, pp. 1-13, M. Mausek, B. Glumac, *Criticality Safety and Sensitivity Analyses of PWR Spent Nuclear Fuel Repository Facilities*, January 2005.
737. NT-2005, **151**, pp. 51-59, J. J. Casal, J. Krouthen, M. Albendea, *Accurate Tools to Model Advanced SVEA Fuel Designs*, July 2005.
738. NT-2005, **151**, pp. 60-69, A. A. Karve et. al., *Uncertainty Estimates in Cold Critical Eigenvalue Predictions*, July 2005.

739. NT-2005, **151**, pp. 70-76, S. Jacobsson, et. al., *Nondestructive Experimental Determination of the Pin-Power Distribution in Nuclear Fuel Assemblies*, July 2005.
740. NT-2005, **151**, pp. 86-96, S. Yilmaz et. al., *Optimizing the Placement of Burnable Poisons in PWRs*, July 2005.
741. NT-2005, **151**, pp. 96, P. H. Wakker, *Reducing Duration of Refueling Outage by Optimizing Core Design and Shuffling Sequence*, July 2005.
742. NT-2005, **151**, pp. 109, J. R. Secker, *Optimum Discharge Burnup and Cycle Lengths for PWRs*, August 2005.
743. NT-2005, **151**, pp. 120, C. Brown, K. Hartley, J. Hulsman, *Extended Power Uprates and 2-Year Cycles for BWRs – Where Do We Go from Here?*, August 2005.
744. NT-2005, **151**, pp. 134, B. T. Rearden, W. J. Anderson, G. A. Harms, *Use of Sensitivity and Uncertainty Analysis in the Design of Reactor Physics and Criticality Benchmark Experiments for Advanced Nuclear Fuel*, August 2005.
745. NT-2005, **151**, pp. 201, S. Kalcheva, E. Koonen, B. Ponsard, *Accuracy of Monte Carlo Criticality Calculations During BR2 Operation*, August 2005.

Nuclear Science and Engineering

746. NSE-1990, **104**, 66, T. L. Sanders, R. M. Westfall, *Feasibility and Incentives for Burnup Credit in Spent Fuel Transport Casks*, 1990.
747. NSE-1990, **118**, 79-90, M. D. DeHart, R. E. Pevey, T. A. Parish, *An Extended Step Characteristic Method for Solving the Transport Equation in General Geometries*, 1990. Grunden för programmet NEWT i SCALE 5.
748. NSE-2002, **141**, 32-45, P. Jansson et. al., *Neutronics Investigations for the Lower Part of a Westinghouse SVEA-96+ Assembly*, 2002.
749. NSE-2002, **141**, 129-139, P. Jansson et. al., *Gamma-Ray Spectroscopy Measurements of Decay Heat in Spent Nuclear Fuel*, 2002.
750. NSE-2002, **141**, 175-189, Z. Xu, M. J. Driscoll, M. S. Kazimi, *Neutron Spectrum Effects on Burnup, Reactivity, and Isotopics in UO_2/H_2O Lattices*, 2002.
751. NSE-2002, **142**, 258-269, B. D. Murphy, R. T. Primm III, *Simulation of Mixed-Oxide and Low-Enriched Uranium Fuel Burnup in a Pressurized Water Reactor and Validation Against Destructive Analysis Results*, 2002.
752. NSE-2003, **144**, 115-128, N. Shinohara et. al., *Validation of Minor Actinide Cross Sections by Studying Samples Irradiated for 492 Days at the Dounreay Prototype Fast Reactor – I: Radiochemical Analysis*, 2003.
753. NSE-2003, **144**, 115-128, K. Tsukimoto et. al., *Validation of Minor Actinide Cross Sections by Studying Samples Irradiated for 492 Days at the Dounreay Prototype Fast Reactor – II: Burnup Calculations*, 2003.
754. NSE-2003, **145**, 196-212, K. R. Elam, B. T. Rearden, *Use of Sensitivity and Uncertainty Analysis to Select Benchmarks for the Validation of Computer Codes and Data*, 2004.
755. NSE-2004, **146**, 340-366, B. L. Broadhead, *Sensitivity- and Uncertainty-Based Criticality Safety Validation Techniques*, 2004.
756. NSE-2005, **151**, 261-273, Z. Xu, M. S. Kazimi, M. J. Driscoll, *Impact of High Burnup on PWR Spent Fuel Characteristics*, 2005.

757. NSE-2005, **151**, 344-347, S. Turner, *Reactivity Effects of Streaming Between Discrete Boron Carbide Particles in Neutron Absorber Panels for Storage or Transport of Spent Fuel*, 2005.

Journal of Nuclear Science and Technology

758. JNST-1994, **31**, 596, K. Suyama, T. Iwasaki, N. Hirakawa, *Analysis of Post Irradiation Experiments in PWRs Using New Nuclear Data*, 1994.
759. JNST-2000, **35**, 240, H. Okuno, K. Suyama, T. Sakai, *A Method to Calculate Sensitivity Coefficients of Reactivity to Errors in Estimating Amounts of Nuclide Found in Irradiated Fuel*, 1998.
760. JNST-2000, **37**, 128-138, K. Okumura et. al., *Validation of a Continuous-Energy Monte Carlo Burn-Up Code MVP-BURN and Its Application to Analysis of Post Irradiation Experiment*, 2000.
761. JNST-2000, **37**, 543-547, K. Oeda et. al., *Calibration of Burnup Monitor Installed in Rokkasho Reprocessing Plant*, 2000.
762. JNST-2000, **37**, 924-933, Y. Ando, K. Nishihara, H. Takano, *Estimation of Spent Fuel Compositions from Light Water Reactors*, 2000.
763. JNST-2002, **39**, 82, K. Suyama, T. Iwasaki, N. Hirakawa, *Analysis of Post Irradiation Experiments in PWRs Using New Nuclear Data*, 2002.
764. JNST-2003, **40**, 433-440, T. Kuroishi, Y. Nomura, *Development of Fission Source Acceleration Method for Slow Convergence in Criticality Analyses by Using Matrix Eigenvalue Applicable to Spent Fuel Transport Cask with Axial Burnup Profile*, 2003.
765. JNST-2003, **40**, 544-551, H. Okuno, *Classification of Criticality Calculations with Correlation Coefficient Method and Its Applications to OECD/NEA Burnup Credit Benchmarks Phase III-A and II-A*, 2003.
766. JNST-2005, **42**, 451-461, A. Hidaka et. al., *Radionuclide Release from Mixed-Oxide Fuel Under High Temperature at Elevated Pressure and Influence on Source Terms*, May 2005.
767. JNST-2005, **42**, 661-669, K. Suyama, H. Mochizuki, *Effect of Neutron Induced Reactions of Neodymium-147 and 148 on Burnup Evaluation*, July 2005.

PATRAM

PATRAM 1989 (Washington D.C.)

768. PATRAM 1989, pp. 763-770, P. D. Clemson, P. R. Thorne, *The Criticality Implications of Taking Credit for Fuel Burn-Up*, February 1989.
769. PATRAM 1989, pp. 771-778, M. C. Brady et. al., *Comparison of Analysis Methods for Burnup Credit Applications*, February 1989.
770. PATRAM 1989, pp. 779-785, T. L. Sanders, W. H. Lake, *Alternatives for Implementing Burnup Credit in the Design and Operation of Spent Fuel Transport Casks*, February 1989.
771. PATRAM 1989, pp. 786-792, P. D. Clemson, M. H. Watmough, *Potential Fuel Damage Within a Shipping Cask Following a Postulated Impact Accident*, February 1989.

772. PATRAM 1989, pp. 793-1836, C. R. Marotta, *Bounding Estimates for Criticality Effects of Unburned LWR Fuel Assembly Tips for Casks Assuming Burnup Credit*, February 1989.

PATRAM 1992 (Yokohama)

773. PATRAM 1992, Vol 1. p. 163, W. H. Lake, *The Use of Burnup Credit for Spent Fuel Cask Design*, September 1992.
774. PATRAM 1992, Vol 1. p. 243, M. C. Brady et. al., *Burnup Credit Issues in Transportation and Storage*, September 1992.
775. PATRAM 1992, Vol 1. p. 267, R. I. Ewing, *Burnup Verification Measurements for Spent Nuclear Fuel*, September 1992.
776. PATRAM 1992, Vol 1. p. 1545, J. K. Boskoven, *Burnup Credit Application in a High-Capacity Truck Cask*, September 1992.

PATRAM 1995 (Las Vegas)

777. PATRAM 1995, p. 1407, M. Zachar, P. Pretesacque, *Burnup Credit in Spent Fuel Transport to COGEMA La Hague Reprocessing Plant*, December 1995.
778. PATRAM 1995, p. 1415, K. Kawakami et. al., *The Use of Gadolinia Credit for Criticality Evaluation of a Spent-Fuel Cask*, December 1995.
779. PATRAM 1995, p. 1423, R. I. Ewing, *Application of a Burnup Verification Meter to Actinide-Only Burnup Credit for Spent PWR Fuel*, December 1995.
780. PATRAM 1995, p. 1430, H. Taniuchi, H. Yasada, *Design of High-Capacity Transport/Storage Cask for High Burnup Fuels*, December 1995.

PATRAM 1998 (Paris)

781. PATRAM 1998 (ORNL/CP-97228) p. 239, L. B. Shappert et. al., *Recent Experience in Planning, packaging and Preparing Noncommercial Spent Fuel for Shipment in the United States*, May 1998.
782. PATRAM 1998, p. 809, W. H. Lake, D. B. Lancaster, *The U.S. Department of Energy's Transportation Burnup Credit Program*, May 1998.
783. PATRAM 1998, p. 817, T. Matsumura et. al., *Analysis of BUrnup Credit on Spent Fuel Transport/Storage Casks – Estimation of Reactivity Bias*, May 1998.
784. PATRAM 1998, p. 825, Y. K. Lee, *A Burnup Credit Calculation Methodology for PWR Spent Fuel Transportation*, May 1998.
785. PATRAM 1998, p. 833, Y. Nomura et. al., *OECD/NEA Working Party on Nuclear Criticality Safety: Challenge of New Relaties*, May 1998.
786. PATRAM 1998, p. 875?, H. Y. Kang et. al., *Package Design for Spent Fuel Used for Reactivity Measurement*, May 1998.
787. PATRAM 1998, p. 88?, T. Kitano, D. Ito, T. Matsumura, *LWR Spent Fuel Cask Reactivity Measurement System*, May 1998.

PATRAM 2001 (Chicago)

788. PATRAM 2001, P. Dyck, *Overview of the Burnup Credit Activities of the IAEA*, September 2001.
789. PATRAM 2001, C. V. Parks et al., *U. S. Regulatory Approach program for Implementation of Burnup Credit in Transport Casks*, September 2001.

790. PATRAM 2001, Y. Nomura et al., *Development of Burnup Credit Evaluation Methods at JAERI*, September 2001.
791. PATRAM 2001, I. Mitsubishi et al., *Accuracy Evaluation of Reactor Records Based on Solution Analyses Data of Spent Fuels at JNC Tokai Reprocessing Plants*, September 2001.
792. PATRAM 2001, M. Doucet et al., *Framatome ANP Capabilities in Nuclear Criticality Safety Studies for Transport Packages and Storage Installations*, September 2001.

PATRAM 2004 (Berlin)

793. PATRAM 2004, T. Saegusa et. al., *Japanese perspectives and research on packaging, transport and storage of spent fuel*, September 2004.
794. PATRAM 2004, paper #110, R. Riersh, F. Thomas, *Use of CASTOR and CONSTOR casks for RBMK and VVER fuel assemblies*, September 2004.
795. PATRAM 2004, paper #116, R. Srinivasan, S. Sisley, J. E. Hopf, *Transporting Existing VSC-24 Canisters Using a Risk-Based Licensing Approach*, September 2004.
796. PATRAM 2004, paper #134, P. C. Purcell, M. Dallongeville, *Testing of LWR Fuel Rods to Support Criticality Safety Analysis of Transport Accident Conditions*, September 2004.
797. PATRAM 2004, paper #154, C. V. Parks, J. C. Wagner, *Status of Burnup Credit for Transport of SNF in the United States*, September 2004.
798. PATRAM 2004, paper #258, N. L. Osgood, C. J. Withee, E. P. Easton, *Criticality Safety of Spent Fuel Casks Considering Water Inleakage*, September 2004.
799. PATRAM 2004, paper #259, G. S. Bjorkman, *The Buckling of Fuel Rods in Transportation Casks Under Hypothetical Accident Conditions*, September 2004.
800. PATRAM 2004, paper #287, A. Zimmer, et. al., *Expansion of the Capabilities of the GA-4 Legal Weight Truck Spent Fuel Shipping Cask*, September 2004.

RAMTRANS

801. RAMTRANS, Vol 1, No.1, pp. 25-32, H. P. Alesso et. al., *Status of Benchmarks for Criticality Calculations Involving Fission Product Poisons and Special Actinides in Spent Fuel*, 1990.
802. RAMTRANS, Vol 1, No.3, pp. 187-199, R. W: Carlson, L. E. Fischer, *Criticality Safety for Spent Fuel Casks*, 1990.
803. RAMTRANS, Vol 5, No.2-4, pp. 273-278, M. Zachar, P. Pretesacque, *Burnup Credit in Spent Fuel Transport to COGEMA La Hague Reprocessing Plant*, 1994.
804. RAMTRANS, Vol 8, No.2, pp. 127-132, C. K. Wilson, *The 1996 IAEA Transport Regulations for the Safe Transport of Radioactive Materials – The Transport of Fissile Material*, 1997.
805. RAMTRANS, Vol 9, No.4, pp. 249-255, E. Fuentes et. al., *Actinide-Only Burnup Credit for Spent Fuel Transport*, 1999.
806. RAMTRANS, Vol 12, No. 4, pp. 213-224, M. C. Brady-Raap, Y. Nomura, E. Sartori, *Overview of the Burnup Credit Activities of the Organization for Economic Cooperation and Development/Nuclear Energy Agency (OECD/NEA)*, 2001.

807. RAMTRANS, Vol 13, No. 2, pp. 93-100, C. Lavarenne et. al., *Current Studies Related to the Use of Burnup Credit in France*, 2002.

JAERI

808. JAERI-Data/Code 96-036, M. Kurosawa et. al., *The Isotopic Compositions Database System on Spent Fuels LWRs (SFCOMPO)*, February 1997.
809. JAERI-Data/Code 97-045, K. Suyama, *Spent Fuel Isotopic Composition Data Base System on WWW-SFCOMPO on W3*, 1997.
810. JAERI-Data/Code 2001-029, Y. Nomura, M. Murazaki, H. Okuno, *Preparation of Data Relevant to "Equivalent Uniform Burnup" and "Equivalent Initial Enrichment" for Burnup Credit Evaluation*, October 2001. Japanska.
811. JAERI-Data/Code 2002-025, T. Nakagawa, O. Iwamoto, *Comparison of Fission and Capture Cross Sections of Minor Actinides*, January 2003.
812. JAERI-M 94-034. Y. Naito, *Data Book of the Isotopic Composition of Spent Fuel in Light Water Reactors*, 1994.
813. JAERI-M 94-003. Se NEA/NSC/DOC(93)/22.
814. JAERI-Research 96-003. Se NEA/NSC/DOC(96)/01.
815. JAERI-Research 2000-0441. Se NEA/NSC/DOC(2000)12
816. JAERI-Research 2002-001. Se NEA/NSC/DOC(2002)2.
817. JAERI-Tech 2000-071, N. Nakahara et. al., *Technical Development on Burn-Up Credit for Spent LWR Fuels*, (In Japanese), 2000. Översatt av ORNL.
818. JAERI-Tech 2001-055, *A Guide Introducing Burnup Credit, Preliminary Version*, July 2001 (Japanska).
819. JAERI-Tech 2003-021, T. Kuroishi, H. A. Tuan, Y. Nomura, et. al., *Extended Calculations of OECD/NEA Phase II-C Burnup Credit Criticality Benchmark Problem for PWR Spent Fuel Transport Cask by Using MCNP-4B2 Code and JENDL-3.2 Library*, 2003.
820. JAERI-Tech 2004-030, Y. Nomura, H. Okuno, Y. Miyoshi, *Criticality Safety Assessment by Assuming Spent Fuel Burnup Distribution – Examination of Various Methods for Setting Burnup (1)*, 2004. Japanska.f

EPRI (Electric Power Research Institute)

EPRI has issued a number of reports concerning burnup credit. They come at a substantial cost (50 000 U.S. dollar in one case). Abstracts are available on the web page of EPRI.

821. EPRI-1003418, Abstract, *Burnup Credit – Technical Basis for Spent-Fuel Burnup Verification*, December 2003. List price: \$50000.
822. EPRI-1002879, Abstract, *Fission Product Benchmarking for Burnup Credit Application: Progress Report*, December 2002. List price: ?
823. EPRI-TR-10929, Abstract, *Determination of the Accuracy of Utility Spent Fuel Burnup Records (Interim Report)*, May 1998. List price: \$10000.
824. EPRI-NP-6494, Abstract, *Evaluation of Burnup Credit for Dry Storage Casks*, August 1989. List price: \$2500.
825. EPRI-TR-100676, Abstract, *Proceedings: 1991 EEI/UWaste-EPRI Workshop on At-Reactor Spent-Fuel Storage*, May 1992. List price: \$2500.

ARIANE and other projects

826. R. T. Primm III (U.S. Coordinator), *ARIANE International Programme Final Report*, ORNL/SUB/97-XSV750-1. Released to members in March 2001 and earlier. Published May 2003.
827. Belgonucleaire, ARIANE, Technical proposal, *An experimental programme to improve the evaluation and the prediction of actinides and fission products in MOX and in UO₂ spent fuel elements*, AR 94/02, Rev. March 1996. In [826].
828. Belgonucleaire, *Programme Status*, 0005241/241. Around 2000. In [826].
829. Belgonucleaire, *ARIANE International Programme, Irradiation Data Report Part 1*, AR 96/05 Rev. B. March 1999. In [826].
830. Belgonucleaire, *ARIANE International Programme, Irradiation Data Report Part 2*, AR 99/13. July 1999. In [826].
831. Belgonucleaire, *ARIANE International Programme, Irradiation Data Report Part 3*, AR 2001/20. February 2001. In [826].
832. Belgonucleaire, *Reassessment of the identity of rods irradiated in assembly M109 and used in the framework of the programme M109, ARIANE and FIGARO*, Technical Note 9904108/221. June 1999. In [826].
833. Belgonucleaire, *ARIANE International Programme, SIMS Analysis of the three irradiated fuel specimens*, AR 99/12. March 1999. In [826].
834. Belgonucleaire, *ARIANE International Programme, Final Report*, AR 2000/15. December 2000. In [826].

Other conferences

WRSI - Water Reactor Safety Information Meeting

835. RWSI-27, pp. 419-436, D. E. Carlson, C. J. Withee, C. V. Parks, *Spent Fuel Burnup Credit in Casks: An NRC Perspective*, NUREG/CP-0169, Appendix ISG 8 rev. 1 and presentation, October 1999.
836. RWSI-27, pp. 437-446, D. Lancaster, W. Lake, A. Machiels, *Burnup Credit for Spent Fuel Transport*, NUREG/CP-0169, October 1999.
837. RWSI-27, pp. 447-458, C. Lavarenne et. al., *Taking Burnup Credit into Account in Criticality Studies: The Situation as It Is Now and the Prospekt for the Future*, NUREG/CP-0169, October 1999.
838. RWSI-27, pp. 459-473, J. Basselier et. al., *The REBUS International Program (Critical Experiment with Spent-Fuel for Burnup-Credit Validation)*, NUREG/CP-0169, October 1999.
839. RWSI-28, C. V. Parks et. al., *Research Supporting Implementation of Burnup Credit in the Criticality Safety Assessment of Transport and Storage Casks*, October 2000.

PHYSOR

There are large amounts of papers that refer to calculation methods used in reactor physics, including burnup credit. Only a selection of those that were found to be directly linked to burnup credit are listed.

PHYSOR 1996 (Mito, Japan)

- 840. PHYSOR 1996, M. C. Brady et. al., *Findings of the OECD/NEA Study on Burnup Credit*, September 1996.
- 841. PHYSOR 1996, N. T. Gulliford, *BUC validation in the UK: design of experiments and lessons learnt*, September 1996.
- 842. PHYSOR 1996, N. T. Gulliford, D. Hanlon, *Measurement of Reactivity with Burnup in PWR and MOX Fuel in the CERES Colaborative Programme*, September 1996.

“PHYSOR” 1998 (Long Island)

- 843. PHYSOR 1998, p. 619, J. M. Conde et. al., *An Overview of International Activities on the Use of Burnup in Spent Fuel Management*, October 1998.
- 844. PHYSOR 1998, p. 624, M. C. Brady et. al., *International Studies on Burnup Credit Criticality Safety by an OECD/NEA Working Group*, October 1998.
- 845. PHYSOR 1998, p. 631, J-C Neuber, *Burnup Credit Applications to PWR and BWR Fuel Assembly Wet Storage Systems*, October 1998.
- 846. PHYSOR 1998, p. 735, D. Lancaster, C. Kang, W. H. Lake, *Status of Burnup Credit in the United States*, October 1998.
- 847. PHYSOR 1998, p. 742, H. Okuno, Y. Nomura, K. Suyama, *Status on Burnup Credit at JAERI*, October 1998.
- 848. PHYSOR 1998, p. 749, X. Boudin, E. Letang, E. Guillou, *Taking Burnup Credit in Account in Current and Future Criticality Studies – French Working Group on Burnup Credit*, October 1998.

PHYSOR 2000 (Pittsburgh)

- 849. PHYSOR 2000, V. A. F. Dean, *Evaluating Experiments for Code and Cross-Section Validation for Criticality Safety*, May 2000.
- 850. PHYSOR 2000, D. Lancaster, *Meeting the NRC Guidance on Burnup Credit*, May 2000.
- 851. PHYSOR 2000, X. Xiaogang et. al., *Feasibility Study for Burnup Credit in Spent-Fuel Storage for Nuclear Power Station*, May 2000.
- 852. PHYSOR 2000, B. D. Murphy, R. T. Primm III, *Prediction of Spent MOX and LEU Fuel Composition with Measurements*, May 2000.
- 853. PHYSOR 2000, C. Chabert et. al., *Qualification of the APOLLO2 Assembly Code Using PWR-UO2 Isotopic Assays. The Importance of Irradiation History and Thermomechanics on Fuel Inventory Prediction*, May 2000.

PHYSOR 2002 (Seoul, Sydkorea)

- 854. PHYSOR 2002, D. Fujiwara, S. Kosaka, *Isotopic Concentration and Criticality Analyses of BWR Spent Nuclear Fuel Using CASMO*, October 2002.

- 855. PHYSOR 2002, B. Lance et. al., *Status of the REBUS-PWR International Programme*, October 2002.
- 856. PHYSOR 2002, H. Okuno, K. Tonoike, T. Sakai, *Burnup Importance Function and Its Application to OECD/NEA/BUC Phase II-A and II-C Models*, October 2002.
- 857. PHYSOR 2002, B. Roque et. al., *Experimental Validation of the Depletion Code System "DARWIN" for Spent Fuel Isotopic Predictions in Fuel Cycle Applications*, October 2002.
- 858. PHYSOR 2002, M. Milosevic, E. Greenspan, J. Vujic, *A SAS2H/KENO-V Methodology for 3D Depletion ANalysis*, October 2002.
- 859. PHYSOR 2002, W. Bernnat et. al., *Monte Carlo Reactor Physics Calculations for Critical Assemblies and LWR Full Core Models*, October 2002.
- 860. PHYSOR 2002, K. S. Smith, J.D. Rhodes, III, *Full-Core, 2-D, LWR Core Calculations with CASMO-4E*, October 2002.

PHYSOR 2004 (Chicago)

- 861. PHYSOR 2004, J-P Hudelot et. al., *OSMOSE: An Experimental Programme for the Qualification of Integral Cross-Sections of Actinides*, April 2004.
- 862. PHYSOR 2004, J. C. Kuijper et. al., *HTR-N Plutonium Cell Burnup Benchmark: Definition, Results & Intercomparison*, April 2004.
- 863. PHYSOR 2004, C. Trakas, L. Daudin, *Benchmark of MONTEBURNS against Measurements on Irradiated UOX and MOX Fuels*, April 2004.
- 864. PHYSOR 2004, A. Courcelle, A. Santamarina, S. Mengelle, *Improvements of Isotopic Ratios Prediction through TAKAHAMA-3 Chemical Assays with the JEFF-3.0 Nuclear Data Library*, April 2004.
- 865. PHYSOR 2004, K. Suyama et. al., *Validation of Integrated Burnup Code System SWAT2 by the Analysis of Isotopic Composition Data of Spent Nuclear Fuel*, April 2004.
- 866. PHYSOR 2004, M. F. Murphy et. al., *Reactivity and Neutron Emission Measurements of Burnt PWR Fuel Rod Samples in LWR-PROTEUS Phase II*, April 2004.

IHLRWM - International High-Level Radioactive Waste Management Las Vegas

IHLRWM-5, May 1994

- 867. IHLRWM-5(1994), p. 732, R. I. Ewing, *Burnup Verification Using the FORK Measurement System*, May 1994.
- 868. IHLRWM-5(1994), p. 739, R. J. Cacciapouti, S. Van Volkinburg, L. A. Hassler, *PWR Axial Burnup Profile Database*, May 1994.
- 869. IHLRWM-5(1994), p. 744, T. A. Parish, *Effects of Axial Burnup Distributions on the Reactivity of Spent Fuel*, May 1994.
- 870. IHLRWM-5(1994), p. 752, C-K. C. Wang, J. W. Foster, D. Napolitano, *Implementation of Response Function Concept for Spent Fuel Cask Analyses*, May 1994.
- 871. IHLRWM-5(1994), p. 757, J. Anno, *Fitted Analytical Expression Giving the Enrichment in U235 of a Fresh Fuel Equal to a Spent PWR Fuel*, May 1994.

- 872. IHLRWM-5(1994), p. 823, J. R. Thornton et. al., *Multi-Purpose Canister Criticality Control Design Features*, May 1994.
- 873. IHLRWM-5(1994), p. 831, T. Suto, S. M. Bowman, C. V. Parks, *The Reactivity Effects of Nuclide Buildup and Decay During Long-Term Fuel Storage*, May 1994.
- 874. IHLRWM-5(1994), p. 838, D. A. Thomas, T. W. Doering, *Long Term Nuclear Criticality Potential in Waste Packages*, May 1994.
- 875. IHLRWM-5(1994), p. 845, P. Gottlieb, J. K. McCoy, *Design Basis Spent Nuclear Fuel Characteristics*, May 1994.
- 876. IHLRWM-5(1994), p. 852, D. Evans, B. Palmer, *Repository Criticality Safety for the DOE Spent Nuclear Fuel Program*, May 1994.

IHLRWM-6, April 1995

- 877. IHLRWM-6(1995), p. 343, A. S. Mobasheran, J. Boshoven, B. Lake, *Impacts of the Use of Spent Nuclear Fuel Burnup Credit on DOE Advanced Technology Legal Weight Truck Cask GA-4 Fleet Size*, April 1995.
- 878. IHLRWM-6(1995), p. 433, S. B. Ludwig, R. B. Pope, *Characterization of Spent MOX Fuel Resulting From Disposition of SFM and Potential Impacts on the CRWMS*, April 1995.
- 879. IHLRWM-6(1995), p. 437, W. G. Halsey et. al., *Comparative Repository Performance of Plutonium Forms – Initial Studies*, April 1995.
- 880. IHLRWM-6(1995), p. 437, W. G. Halsey et. al., *Comparative Repository performance of Plutonium Forms – Initial Studies*, April 1995.
- 881. IHLRWM-6(1995), p. 541, R. J. Cacciapouti, S. Van Volkinburg, *Update on the PWR Axial Burnup Profile Database*, April 1995.
- 882. IHLRWM-6(1995), p. 544, R. W. Leary, T. A. Parish, *Reactivity Effects of Nonuniform Axial Burnup Distributions on Spent Fuel*, April 1995.
- 883. IHLRWM-6(1995), p. 547, D. A. Thomas, T. W. Doering, *Long-Term Criticality Analysis Process*, April 1995.
- 884. IHLRWM-6(1995), p. 553, L. C. Leal, O. W. Hermann, C. V. Parks, *ARP: A PC-Compatible Scheme for Generating ORIGEN-S Cross Section Library*, April 1995.
- 885. IHLRWM-6(1995), p. 556, R. I. Ewing, G. E. Bosler, J. Priore, *Burnup Verification at Arkansas Nuclear One-Unit 1 Using the FORK Measurement System*, April 1995.
- 886. IHLRWM-6(1995), p. 588, J. K. Bates et. al., *Reactivity of Plutonium-Containing Glasses for the Immobilization of Surplus Fissile materials*, April 1995.

IHLRWM-7, April 1996

- 887. IHLRWM-7(1996), p. 330, A. S. Mobasheran, B. Lake, J. Richardson, *Impacts of SNF Burnup Credit on the Shipment Capability of the GA-4 Cask*, April 1996.
- 888. IHLRWM-7(1996), p. 333, B. Snyder, *Dense Organic Liquids Reduce GA-4 Reactivity Margin*, April 1996.
- 889. IHLRWM-7(1996), p. 336, Ch-H Chen, T. A. Parish, *Bounding Axial Profile Analysis for the Topical Report Database*, April 1996.
- 890. IHLRWM-7(1996), p. 340, R. I. Ewing, K. D. Seager, *Design of an Advanced FORK System for Assembly Burnup Measurement*, April 1996.

- 891. IHLRWM-7(1996), p. 342, R. C. Ashline, C. W. Forsberg, *U.S. Light Water Reactor Spent Fuel Inventory – Fissile Distribution*, April 1996.
- 892. IHLRWM-7(1996), p. 345, P. Gottlieb, J. R. Massari, *Probabilistic Evaluation of Postclosure Criticality Events Internal to the Waste Package*, April 1996.
- 893. IHLRWM-7(1996), p. 348, M. A. Rodriguez, *A Follow-Up Study to: Job Performance Aids to Criticality Safety*, April 1996.

IHLRWM-8, May 1997

- 894. IHLRWM-8(1997), p. 371, M. Scott, P. Russell, T. Doering, *Rethinking Regulations for Disposal Criticality*, May 1997.
- 895. IHLRWM-8(1997), p. 450, L. Gratton et. al., *Phenomena Affecting the Dynamics of Critical Deposits in Tuff*, May 1997.
- 896. IHLRWM-8(1997), p. 567, R. J. Cacciapouti, S. Van Volkinburg, *Axial Burnup Profile Database for Pressurized Water Reactors*, May 1997.
- 897. IHLRWM-8(1997), p. 569, C. H. Kang, D. B. Lancaster, *Conservative Axial Burnup Distributions for Actinide-Only Burnup Credit*, May 1997.
- 898. IHLRWM-8(1997), p. 573, K. D. Wright, *Commercial Reactor (PWR) Critical Evaluations for Benchmarking SAS2H and MCNP4A*, May 1997.
- 899. IHLRWM-8(1997), p. 576, D. P. Henderson, M. J. Anderson, *BWR Commercial Reactor Critical Benchmark Calculations Using SAS2H and MCNP*, May 1997.
- 900. IHLRWM-8(1997), p. 579, M. Rahimi, *Isotopic Validation for PWR Actinide-Only Burnup Credit Using Yankee Rowe Data*, May 1997.
- 901. IHLRWM-8(1997), p. 583, D. B. Lancaster, E. Fuentes, C. Kang, *Method for Adding Additional Isotopes to Actinide-Only Burnup Credit*, May 1997.
- 902. IHLRWM-8(1997), p. 627, J. R. Massari, P. Gottlieb, *Criticality Potential of Commercial PWR SNF In a Degraded Waste Package*, May 1997.
- 903. IHLRWM-8(1997), p. 630, J. A. McClure, J. R. Worsham III, *RELAP5 Analysis of Postulated Criticality Events In Degraded Waste Packages*, May 1997.
- 904. IHLRWM-8(1997), p. 633, J. W. Davis, P. Gottlieb, *Disposal Criticality Analysis for Aluminum-Based DOE Fuels*, May 1997.
- 905. IHLRWM-8(1997), p. 636, C. W. Forsberg, *In-Repository Depleted-Uranium Isotopic Mixing with LWR SNF for Criticality Control*, May 1997.

IHLRWM-9, April-May 2001

- 906. IHLRWM-9(2001), J. Richardson, *Impacts of the Use of Spent Nuclear Fuel Burn Up Credit on Advanced Legal Weight Truck Casks*, May 2001.
- 907. IHLRWM-9(2001), C. V. Parks, R. Y. Lee, *Research Supporting Implementation of Burnup Credit in the Criticality Safety Assessment of Transport and Storage Casks*, May 2001.

IHLRWM-10, March-April 2003

- 908. IHLRWM-10(2003), pp. 443-449, S. F. Kessler, *Criticality Safety of Hanford Spent Nuclear Fuel Storage*, April 2003.
- 909. IHLRWM-10(2003), pp. 458-464, P. Dybeck, *Transport of Encapsulated Spent Fuel to a Final Repository in Sweden*, April 2003.
- 910. IHLRWM-10(2003), pp. 500-507, C. V. Parks, C. J. Withee, *Recommendations for PWR Storage and Transportation Casks That Use Burnup Credit*, April 2003.

911. IHLRWM-10(2003), pp. 733-739, W. L. Hurt et. al., *Development of Neutron Absorbers to Support Disposal of DOE SNF*, April 2003.
912. IHLRWM-10(2003), pp. 903-909, O. Wantz et. al., *In-Package Criticality Scenarios Development for the Belgian Spent Fuel Repository*, April 2003.
913. IHLRWM-10(2003), pp. 917-924, B. Kienzler et. al., *Is Criticality a Matter of Concern for Gorleben?*, April 2003.
914. IHLRWM-10(2003), pp. 925-931, M. Rahimi, J. Weldy, *Estimating In-Package Criticality Impacts on Yucca Mountain Repository Performance*, April 2003.

INMM - International Nuclear Materials & Management

915. INMM-36(1995)-70, G. A. Harms, F. J. Davis, J. T. Ford, *The Spent Fuel Safety Experiment*, July 1995.
916. INMM-36(1995)-84, R. I. Ewing, *Burnup Verification Measurements On Spent Fuel Assemblies at Arkansas Nuclear One*, July 1995. (samma som ICNC 1995)
917. INMM-46(2005)-157, M. Okano et. al., *Development of Analysis Method for Plutonium amount and Burn up by Measurement of Xenon Isotopic Ratio in Dissolver Off-gas at Reprocessing Facility*, July 2005.
918. INMM-46(2005)-352, S. E. Sisley, R. D. Quinn, *Burnup Credit Misload Evaluation of Loaded Storage Canisters*, July 2005.

NUCEF (JAERI, Tokaimura)

919. NUCEF-1998 (JAERI-Conf 99-004), p. 61, R. T. H. Mayson, K. J. Guston, *Challenges in the Application of Burnup Credit to the Criticality Safety of the THORP Reprocessing Plant*, March 1999.
920. NUCEF-1998 (JAERI-Conf 99-004), p. 68, H .S. Shin et. al, *Application of Burnup Credit for PWR Spent Fuel Storage Pool*, March 1999.
921. NUCEF-1998 (JAERI-Conf 99-004), p. 77, K.Suyama, J. Inagawa, M. Murazaki, *Analysis of Reactivity Effect of Fission Products*, March 1999.
922. NUCEF-2001 (JAERI-Conf 2002-004), p. 81, Y. Nomura, H. Okuno, K. Suyama, *Development of Burnup Credit Evaluation Methods at JAERI*, March 2002.
923. NUCEF-2001 (JAERI-Conf 2002-004), p. 91, P D'hondt et. al., *The REBUS Experimental Programme for Burn-up Credit*, March 2002.
924. NUCEF-2001 (JAERI-Conf 2002-004), p. 391, T. Hayashi, Y. Yamane, K. Suyama, *Reactivity Effect of Spent Fuel Due to Spacial Distribution of Coolant Temperature and Burnup*, March 2002.
925. NUCEF-2001 (JAERI-Conf 2002-004), p. 399, H. A. Tuan, H. Okuno, Y. Nomura, *Reactivity Effect of Axial Burnup Shapes for a Realistic PWR Spent Fuel Transport Cask*, March 2002.
926. NUCEF-2001 (JAERI-Conf 2002-004), p. 415, T. Kiyosumi, Y. Miyoshi, *The Calculation Results of OECD/NEA Source Convergence Benchmark Problem No. 2 and No. 4 by Using MCNP*, March 2002.
927. NUCEF-2005 (JAERI-Conf 2005-007), p. 165, D. Lancaster, *2005 Status and Future of Burnup Credit in the USA*, February 2005.

IAEA Conference on Storage of Spent Fuel

928. IAEA-CN-102/19, J-C Neuber, *Use of burnup credit in criticality safety design analysis of spent fuel storage system*, IAEA Storage of Spent Fuel Conference, June 2003.
929. IAEA-CN-102/58, K. Suyama et. al., *SFCOMPO: A database for isotopic composition of nuclear spent fuel – Current status and future development*, IAEA Storage of Spent Fuel Conference, June 2003.
930. IAEA-CN-102/45P, B. Kurincic, A. Persic, *The NPP Krsko reracking project*, IAEA Storage of Spent Fuel Conference, June 2003.
931. IAEA-CN-102/31, K. S. Seo et. al., *Assessment of the storage concept for conditioned spent fuel*, IAEA Storage of Spent Fuel Conference, June 2003.

Various conferences, technical meetings and seminars

932. J. T. Mihalczko et. al., *Feasibility of Subcriticality and NDA Measurements for Spent Fuel by Frequency Analysis Techniques with ^{252}Cf* , ANS International Topical Meeting on Nuclear Plant Instrumentation, Control, and Human-Machine Interface Technologies, 6-9 May 1996.
933. DOE/SNF-FMM-2000, p. 214, R. W. Schaefer, *Criticality Safety Issues in the Disposition of BN-350 Spent Fuel*, DOE Spent Nuclear Fuel and Fissile Material Management, June 2000.
934. DOE/SNF-FMM-2000, p. 246, J. W. Sterbentz, *Spent Nuclear Fuel Source Term Calculational Methodology Used at the Idaho National Engineering and Environmental Laboratory*, DOE Spent Nuclear Fuel and Fissile Material Management, June 2000.
935. DOE/SNF-FMM-2000, p. 253, R. D. McKnight, *ANL Calculational Methodologies for Determining Spent Nuclear Fuel Source Term*, DOE Spent Nuclear Fuel and Fissile Material Management, June 2000.
936. DOE/SNF-FMM-2000, p. 379, R. D. McKnight, J. R. Krsul, *Validation Results Based on the Spent Fuel Demonstration Program at FCF*, DOE Spent Nuclear Fuel and Fissile Material Management, June 2000.
937. GRS/1997-Vortrag, W. Weber, P. Cousinou, *Consideration of Burnup Rates in the Analysis of the Safety of the Nuclear Fuel Cycle*, October 1997.
938. IYNC-2002, C. E. Sanders, M. D. DeHart, *Computational Benchmark of the 2-D Depletion Sequence SAS2D for Characterization of Spent Nuclear Fuel*, April 2002.
939. KAIF/SFM-2002, A. S. Chesterman, *Spent Fuel Measurements to Improve Storage and Transport Efficiency*, KAIF Meeting Spent Fuel Management, Sydkorea, 2002.
940. LWR-FP-2004/1, H.-U. Zwicky et. al., *Isotopic Analysis of Irradiated Fuel Samples in the Studsvik Hotcell Laboratory*, LWR Fuel Performance Meeting, Orlando, September 2004.
941. LWR-FP-2004/2, pp. 273-283, D. Boulanger et. al., *High Burnup PWR and BWR MOX fuel performance: A review of BELGONUCLEAIRE Recent Experimental Programs*, LWR Fuel Performance Meeting, Orlando, September 2004.
942. MC-2005, J. P. Finch, J. S. Hendricks, C. K. Choi, *Vacation Matrix Method for Correct Source Distribution in Monte Carlo Criticality Calculations*, April 2005.
943. MC-2005, H. J. Shim, C. H. Kim, *Convergence Criterion of Fundamental Mode Fission Source Distribution in Monte Carlo Criticality Calculations*, April 2005.

- 944. MC-2005, S. M. Bowman, M. D. DeHart, L. M. Petrie, *Integrated KENO Monte Carlo Transport for 3-D Depletion with SCALE*, April 2005.
- 945. PBC-14/2004, C. V. Parks, J. C. Wagner, *Current Status and Potential Benefits of Burnup Credit Spent Fuel Transportation*, 14th Pacific Basin Conf, March 2004.
- 946. WM'01-1, P. M. O'Leary, M. L. Pitts, *Effects of Burnable Absorbers on PWR Spent Nuclear Fuel*, February 2001. "A Regulatory Analysis of Incidental Waste"
- 947. WM'01-2, W. J. Anderson, *Lacking Spent Nuclear Fuel Critical Benchmarks? – Got Reactor Criticals?*, February 2001.
- 948. M. Hamasaki, *Request from Nuclear Fuel Cycle and Criticality Safety Design*, NDS Japan, 2004.

Other journals

- 949. JKNS-1986-1, B-J Jun, C-K Lee, *A Determination of Bias between Computational Methods for the Criticality Safety Analysis of Spent Fuel Storage Pool with Burnup Credit*, Journal of the Korean Nuclear Society, Vol. 18, No.1, pp. 17-26. March 1986.

Other reports

- 950. ANL/TD/TM00-11, P. Finck, T. Taiwo, J. Gulliford, *Potential Sources for Experimental Validation for Burnup Credit*, January 2000.
- 951. ANL-FRA-2000-1, G. Palmiotti et al., *ANL Pre Analysis of the SHEBA/CERES Experiments*, January 2000.
- 952. ANL-FRA-1998-1, P. J. Finck et al., *Evaluation of Fission Product Worth Margins in PWR Spent Fuel Nuclear Fuel Burnup Credit Calculations*, 1998.
- 953. BCAV-LTRP-060, T. L. Sanders, *Notes on the CSDP Burnup Credit Workshop*, August 1990.
- 954. BNFL/CSM(89)P1-CR-667, A. M. Evans, *Taking Credit for Burnup in Criticality Safety Assessments – A review*,
- 955. CEA/CIefs-45, p. 59, *Spent Fuel Criticality: how to make the most of burnup credit*, March 2002.
- 956. CSFM-87-1, W. C. Morgan, *Criticality Aspects of Rod Consolidation*, September 1987.
- 957. IRSN_DSU/SEC/T/2003.021-SRNC/03.04, G. Poullot et. al., *Catalogue et Historique des Experiences de Criticité Saclay (1958-1964) VALDUC/Batiment 10 (1964-2003)*, October 2003.
- 958. LANL-11096-MS, P. M. Rinard, G. E. Bosler, *Safeguarding LWR Spent Fuel with the FORK Detector*, September 1993.
- 959. M. C. Brady, *Summary of Axial Studies in Generic Cask Model*, July 1989.
- 960. ORNL/TM-2004/124, F. A. Alpan, M. E. Dunn, *YUMMY: The Yucca Mountain MCNP-Library*, October 2004.
- 961. PNL-5109-105, R. J. Guenther et. al., *Characterization of Spent Fuel Approved Testing Material – ATM-105*, December 1991.
- 962. PNL-10045, S. R. Bierman, R. J. Talbert, *Benchmark Data for Validating Irradiated Fuel Compositions Used in Criticality Calculations*, October 1994.
- 963. PNNL-13677, M. C. Brady-Raap, R. J. Talbert, *Compilation of Radiochemical Analyses of Spent Nuclear Fuel Samples*, September 2001.

964. R. I. Ewing, Scientific Notebook Number: BUC18-1-1, *Measurements on Spent Fuel Assemblies at Arkansas Nuclear One (Unit 2 only) Using the Fork System*, September 1995.
965. SAND87-0151, T. L. Sanders, R. M. Westfall, R. H. Jones, *Feasibility and Incentives for Consideration of Spent Fuel Operating Histories in the Criticality Analysis of Spent Fuel Shipping Casks*, August 1987.
966. SAND91-7026, D. G. Adli, D. G. Napolitano, *Burnup Credit Using Advanced Nodal Techniques*, March 1992.
967. SAND92-7066, R. B. Kidman, *MCNP Criticality Calculations of the Burn-up Credit Fresh Fuel Benchmarks*, March 1993.
968. SAND93-2479, R. I. Ewing, G. E. Bosler, G. R. Walden, *Burnup Verification Measurements on Spent Fuel Assemblies at Oconee Nuclear Station*, 1993.
969. SAND2002-3868, E. J. Parma, *BURNAL: A Nuclear Reactor Burnup Code Using MCNP Tallies*, November 2002.
970. SAND2002-3960P, G. A. Harms et. al., *Rhodium Experiments in the NERI Burnup Credit Critical Experiment*, November 2002. Presentationsmaterial (föredrag ANS Winter Meeting 2002).
971. SAND2004-0912, G. A. Harms et. al., *Experimental Investigation of Burnup Credit for Safe Transport, Storage and Disposal of Spent Nuclear Fuel*, April 2004.
972. SLTR-95-0004, P. J. Cooper, *A Demonstration of Burnup Credit Implementation*, April 1995.
973. UCRL-15575, B. H. Warren, M. A. Capo, W. C. O'Neal, *Nuclear Criticality Safety Analysis of a Spent Fuel Waste Package In a Tuff Repository*, December 1983.
974. UCRL-TR-212202, L. J. Jardine (Project leader), *Radiochemical Assays of Irradiated VVER-440 Fuel for Use in Spent Fuel Burnup Credit Activities*, April 2005.
975. USCD 1298/01, R. J. Cacciapouti, S. Van Volkinburg, *Axial Burnup Profile Database for Pressurized Water Reactors*, May 1997. Yankee Atomic, CD-R från OECD/NEA.
976. WAPD-TM-333, T. R. England, *Time-Dependent Fission-Product Thermal and Resonance Absorption Cross Swctions*, November 1982.
977. WSRC-MS-2003-00863, D. A. Eghbali, *Use of Burnup credit as a Safety Factor in Handling of NIST Fuel Assemblies in the L Basin of SRS*, 2003.
978. W. Weber, P. Cousinou, *Consideration of Burnup Rates in the Analysis of the Safety of the Nuclear Fuel Cycle*, GRS, October 1997.
979. YAEC-1937. See USCD 1298/01.
980. YAEC/Memorandum/1990-2907, *Progress Report on Burnup Credit Analysis*, July 1990.

Burnup credit in Sweden

981. SKB R-99-52, L. Agrenius, *Kriticitetsförhållanden i kapslar för slutförvaring av använt kärnbränsle*, September 1999.
982. SKB TR-02-17, L. Agrenius, *Criticality safety calculations of storage canisters*, April 2002.
983. SKI Report 00:13, T. Hicks, A. Prescott, *A Study of Criticality in a Spent Fuel Repository Based on Current Canister Designs*, January 2000.
984. KBS/TR-108, P. Behrens, K. Hannerz, *Criticality in a spent fuel rock repository in wet crystalline rock*, May 1978.