

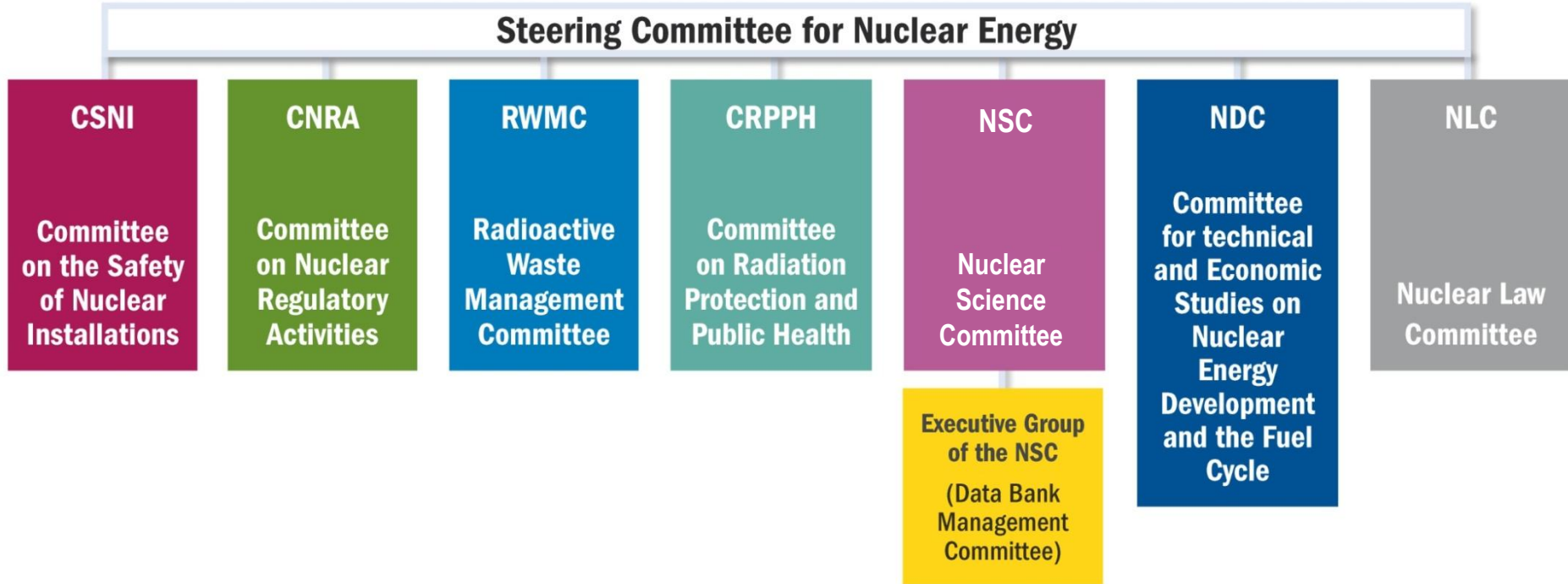
Status of NEA Nuclear Science activities related to accident tolerant fuels

Jim Gulliford, Head of Nuclear Science
OECD-NEA

Outline

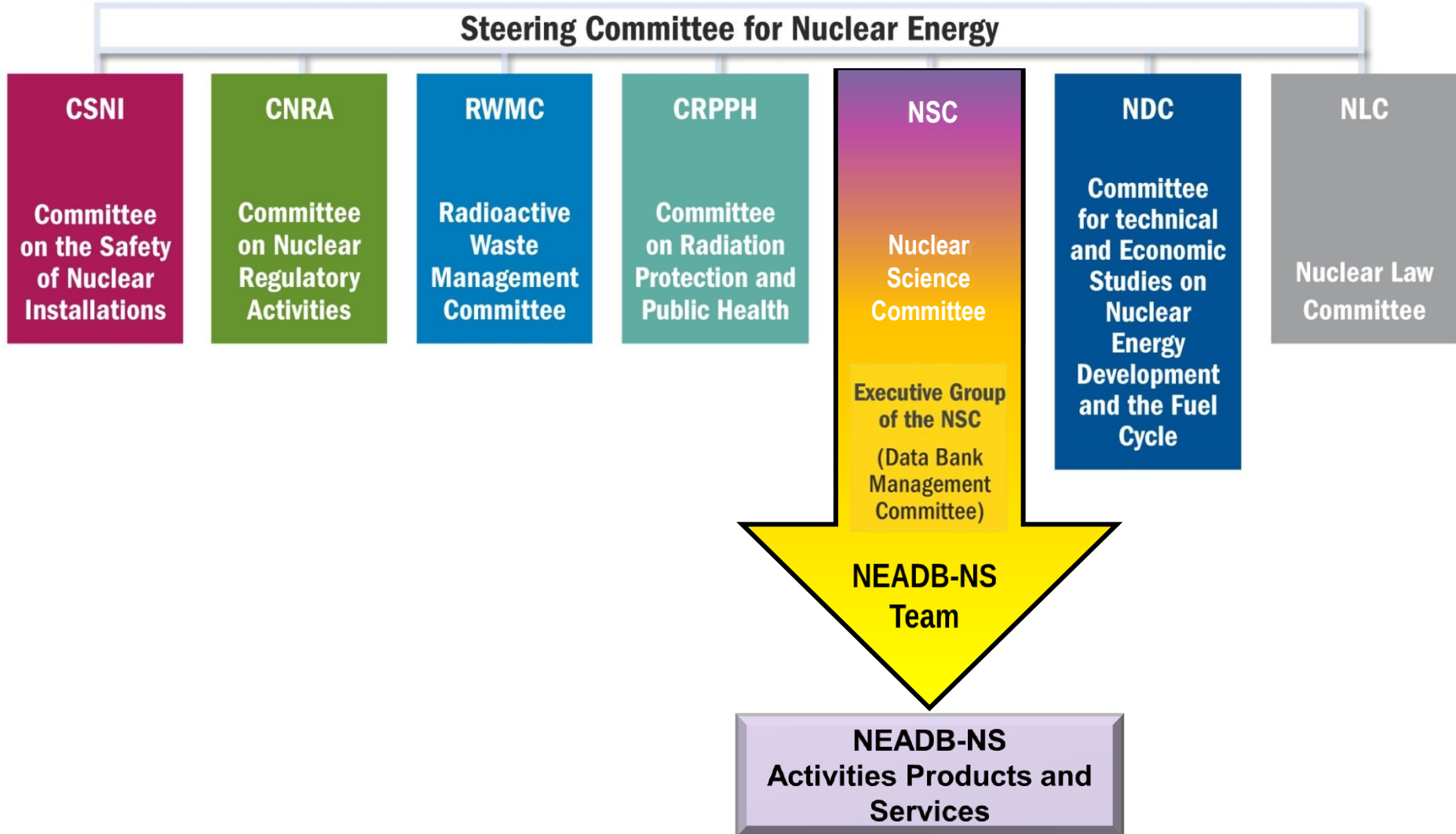
- **OECD-NEA Nuclear Science & Data Bank**
 - **Activities related to innovative fuels and fuels performance**
- **Nuclear Science Activity on ATF for LWRs (EGATFL)**
 - **Response to Fukushima**
 - **Nuclear Science Workshops on ATF**
 - **Initiation of Programme of Work under New Expert Group**

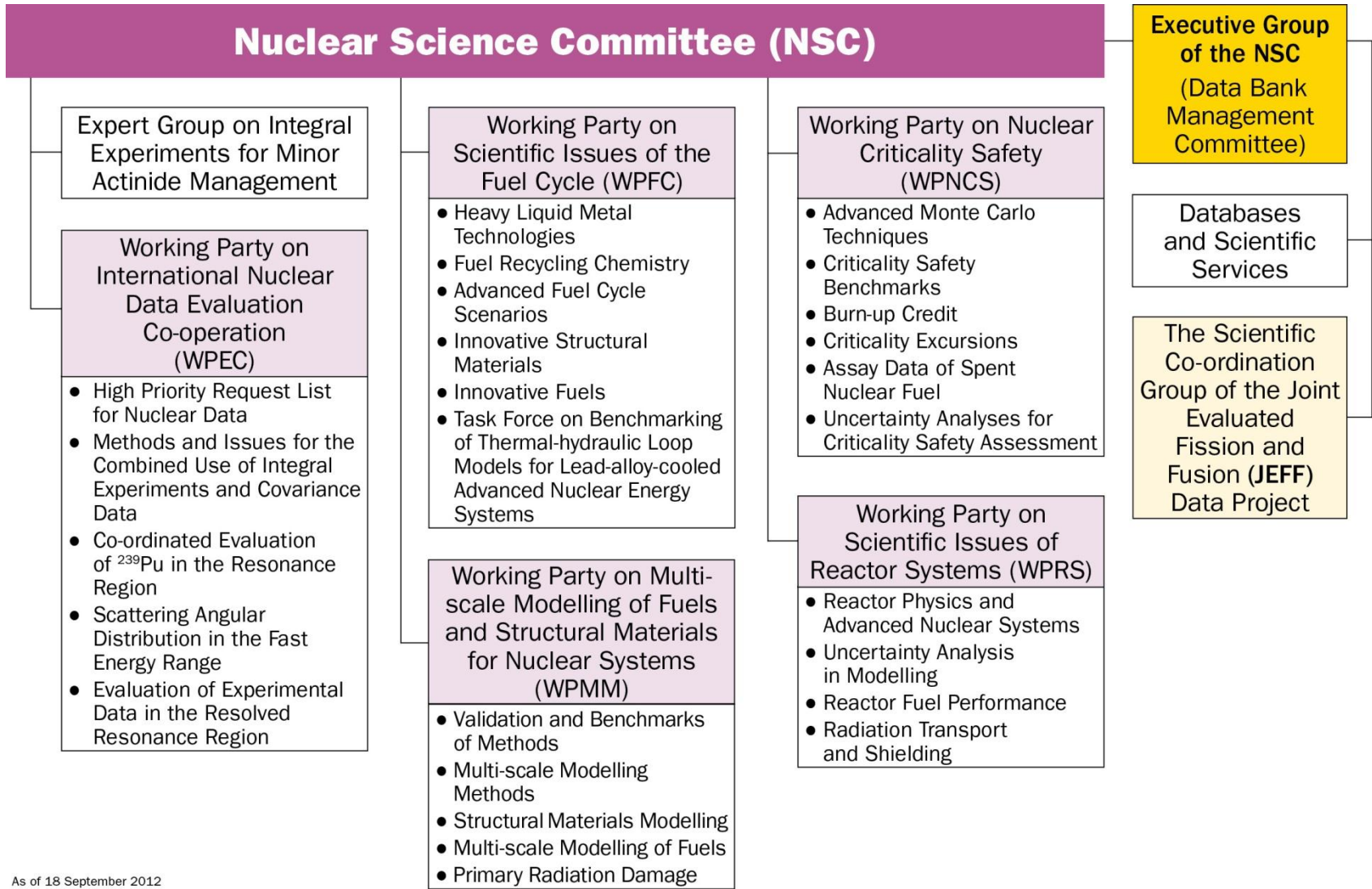
NEA Committees



As of 18 September 2012

Nuclear Science-Data Bank Products & Services





Nuclear Science Committee (NSC)

Expert Group on Integral Experiments for Minor Actinide Management

Working Party on International Nuclear Data Evaluation Co-operation (WPEC)

- High Priority Request List for Nuclear Data
- Methods and Issues for the Combined Use of Integral Experiments and Covariance Data
- Co-ordinated Evaluation of ^{239}Pu in the Resonance

Working Party on Scientific Issues of the Fuel Cycle (WPFC)

- Heavy Liquid Metal Technologies
- Fuel Recycling Chemistry
- Advanced Fuel Cycle Scenarios
- Innovative Structural Materials
- Innovative Fuels
- Task Force on Benchmarking of Thermal-hydraulic Loop Models for Lead-alloy-cooled Advanced Nuclear Energy Systems

Working Party on Nuclear Criticality Safety (WPNCs)

- Advanced Monte Carlo Techniques
- Criticality Safety Benchmarks
- Burn-up Credit
- Criticality Excursions
- Assay Data of Spent Nuclear Fuel
- Uncertainty Analyses for Criticality Safety Assessment

Working Party on Scientific Issues of Reactor Systems (WPRS)

- Reactor Physics and Advanced Nuclear Systems
- Uncertainty Analysis in Modelling
- Reactor Fuel Performance
- Radiation Transport and Shielding

Working Party on Multi-scale Modelling of Fuels and Structural Materials for Nuclear Systems (WPMM)

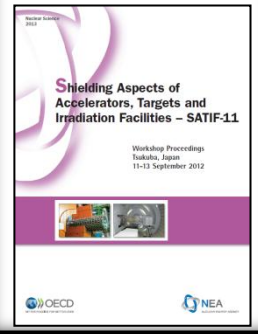
- Validation and Benchmarks of Methods
- Multi-scale Modelling Methods
- Structural Materials Modelling
- Multi-scale Modelling of Fuels
- Primary Radiation Damage

Executive Group of the NSC (Data Bank Management Committee)

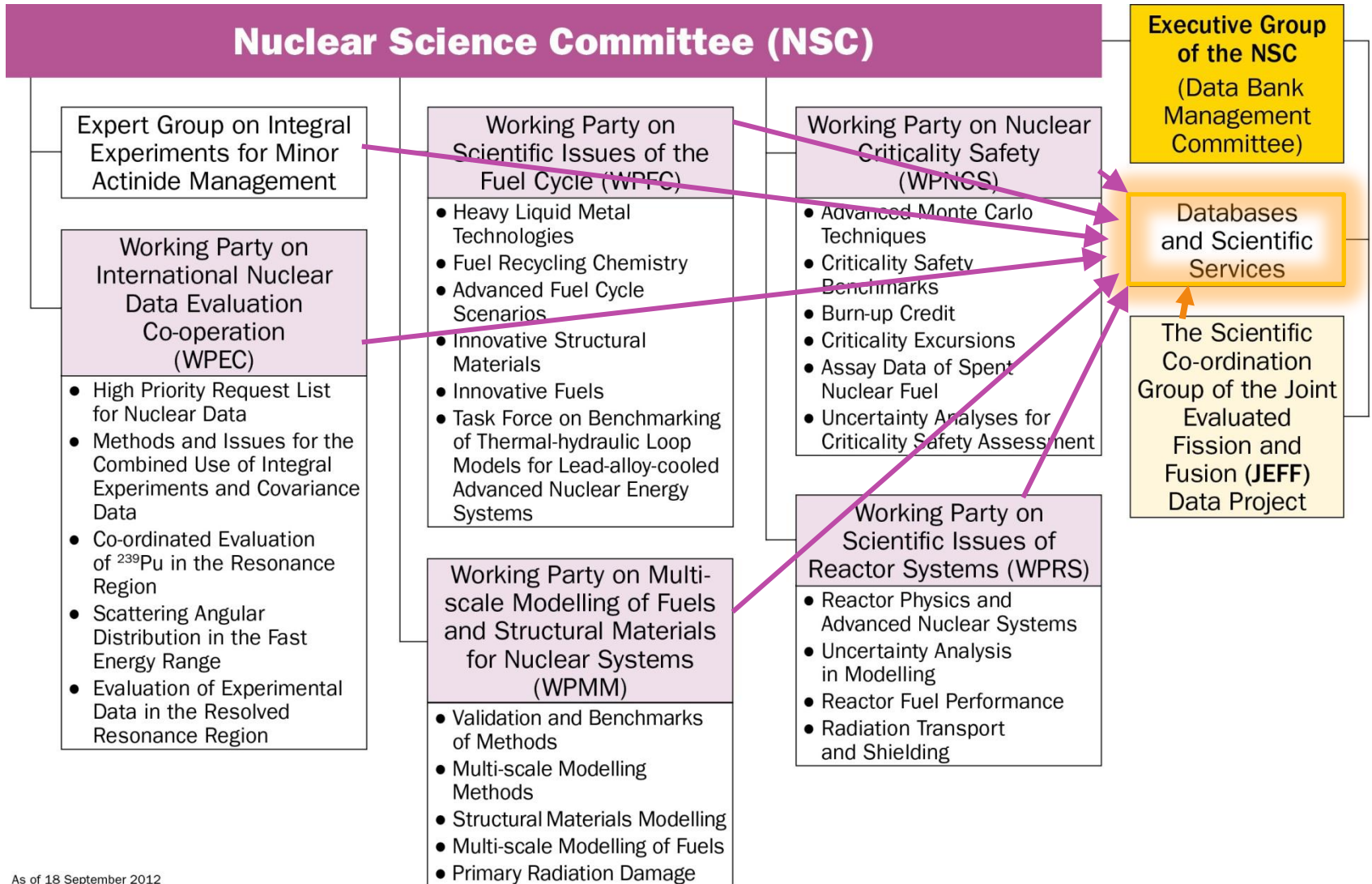
Databases and Scientific Services

The Scientific Co-ordination Group of the Joint Evaluated Fission and Fusion (JEFF) Data Project

Nuclear Science Publications



Angular
in the Fast
range
of Experimental
Resolved
the Region



Nuclear Science Committee (NSC)

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- High Priority Request List for Nuclear Data
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- Scattering Angular Distribution in the Fast Energy Range
- Evaluation of Experimental Data in the Resolved Resonance Region

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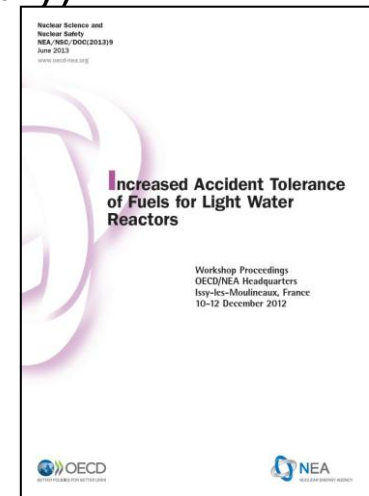
Databases and Scientific Services

The Scientific Co-ordination Group of the Joint Evaluated Fission and Fusion (JEFF) Data Project

Expert Group on Accident Tolerant Fuels for LWRs

Background to Formation of New NEA EGATFL

- **June 2011 Ministerial Meetings, International Fukushima Forum**
- **June 2011 Nuclear Science Meeting – NSFF e-forum**
- **November 2011 Nuclear Science Bureau**
 - Discussion threads from NSFF reviewed
 - Fukushima identified as topic of in-depth discussion at 2012 NSC meeting
- **June 2012 Nuclear Science Meeting**
 - Proposal for ATF Workshop received from the Working Party on Scientific Issues of the Fuel Cycle (K. Pasamehmetoglu, K. McCarthy)
- **10-12 December 2012: 1st OECD-NEA Workshop on Increased Accident Tolerance of Fuels for LWRs**
- **28-29 October 2013: 2nd OECD-NEA Workshop on Increased Accident Tolerance of Fuels for LWRs**
- **28-29 April 2014: Start-up meeting of EGATFL**
- **23-25 September 2014: 2nd EGATFL meeting**



Definition of Accident Tolerant Fuels

Nuclear Energy

Fuels with enhanced accident tolerance are those that, in comparison with the standard UO_2 – Zircaloy system, can **tolerate loss of active cooling** in the core for a **considerably longer time period** (depending on the LWR system and accident scenario) while maintaining or improving the fuel performance during normal operations.

Improved Reaction Kinetics with Steam

- Heat of oxidation
- Oxidation rate

Slower Hydrogen Generation Rate

- Hydrogen bubble
- Hydrogen explosion
- Hydrogen embrittlement of the clad

High temperature
during loss of
active cooling

Improved Fuel Properties

- Lower operating temperatures
- Clad internal oxidation
- Fuel relocation / dispersion
- Fuel melting

Improved Cladding Properties

- Clad fracture
- Geometric stability
- Thermal shock resistance
- Melting of the cladding

Enhanced Retention of Fission Products

- Gaseous FPs, Solid/liquid FPs

Key Elements of PoW for New NEA Expert Group

- **Data and characteristics of candidate materials**, including:
 - Advanced claddings: coated Zr-based alloys, SiC/SiC ceramic composites, advanced steels, refractory metals (e.g. molybdenum), etc.;
 - Advanced Fuels: doped UO₂ for enhanced thermo-mechanical properties, high density fuels such as U-silicide and U-nitride, dispersion fuels with coated particles, etc.;
 - Non-fuel core components such as fuel channels, control rods and blades, and fuel assembly hardware;
- Issues related to the **modelling of the advanced materials** (fuel/cladding behaviour in normal and transient conditions, including DBA and BDBA, etc.);
- A review of **the needs related to an experimental validation** of the most promising materials: available facilities, opportunities of joint experiments (including out-of-pile and in-pile experiments), identification of gaps, etc.
- The **establishment of appropriate metrics** to help prioritise between the ATF candidates;
- The definition and **evaluation of reference scenarios** to evaluate the effectiveness of ATF candidates.

EGATFL Structure

Expert Group on Accident Tolerant Fuels of LWRs (EGATFL)

Information Exchange and Technical Discussions on general issues of ATF

Chair:
K. Pasamehmetoglu
(INL)

Task Force 1 Systems Assessment

- Metrics
 - Economics
 - Fuel cycle (SNF, ...)
 - Operations
 - DBAs
 - BDBAs
 - ...
- TRL definition
- Illustrative Scenario (to feed TF1 and TF2)
- Parametric studies
- System codes

Deliverables

- Definition of illustrative scenarios and first parametric studies
- State-of-the-art report (metrics, system codes)+ Recommendations and priorities

Chair:
S. Bragg-Sitton (INL)

Task Force 2 Cladding/Core Materials

- Properties
- Evaluation under normal operations (incl. fretting)
- Evaluation under illustrative scenario
- PCI
- Testing needs – data gaps (to be linked to RTFDB)
- Modeling needs – modeling gaps
- Experimental infrastructure

Deliverables

- State-of-the-art report + Recommendations and priorities

Chair:
M. Moatti (EDF)

Task Force 3 Fuel Concepts

- Properties
- Evaluation under normal operations (incl. fretting)
- Evaluation under illustrative scenario
- PCI
- Testing needs – data gaps (to be linked to RTFDB)
- Modeling needs – modeling gaps
- Recommendations on priorities

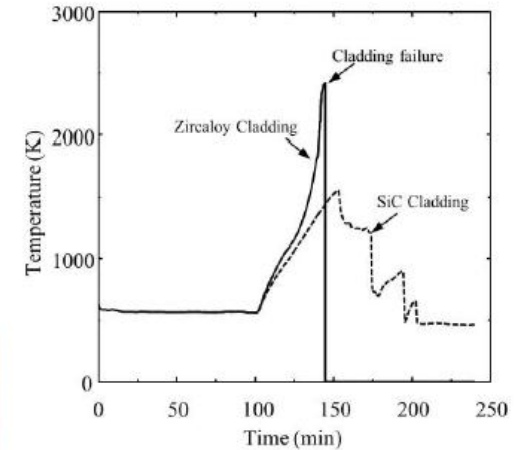
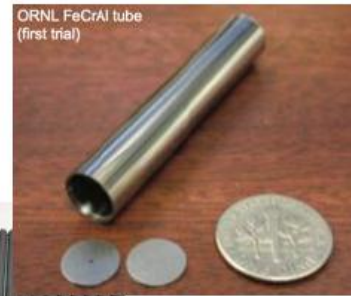
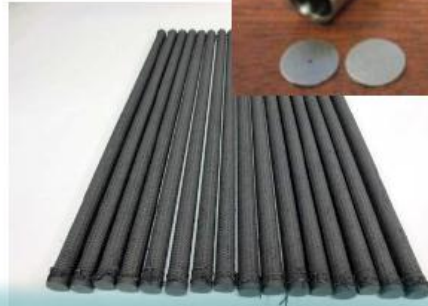
Deliverables

- State-of-the-art report + Recommendations and priorities

Chair:
M. Kurata (JAEA)

ATF candidates: Cladding

- Advanced steels (e.g. FeCrAl)
- Refractory metals (e.g. Mo)
- Ceramic cladding (SiC)
- Innovative alloys with dopants
- Zircaloy with coating or sleeve
 - SiC CMC
 - MAX-phase ceramics
 - Other



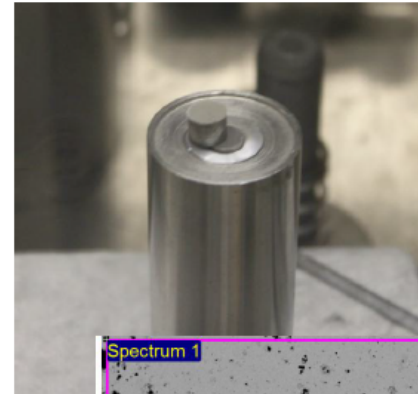
Comparison of MELCOR predicted cladding oxidation heating produced during a TMI-2 accident sequence.

Each concept has some pros and cons across the spectrum of operating and transient conditions of interest. A systematic analytical and experimental evaluation is being performed during the feasibility studies.

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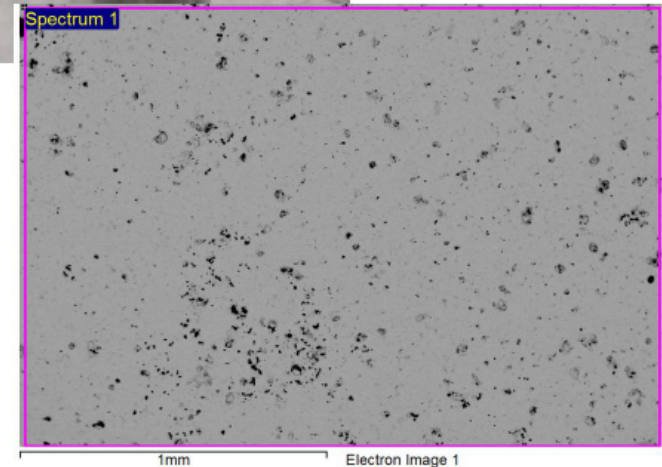
ATF candidates: Fuel

- **Higher density fuels (metal, nitride, silicide)**
 - Higher thermal conductivity
 - Higher fissile density to compensate for neutronic inefficiency of some new cladding concepts without increasing enrichment limits
- **Oxide fuels with additives**
 - Higher thermal conductivity
 - Fission product gettering
- **Microencapsulated fuels**
 - Particle fuel dispersed in a ceramic or metallic matrix



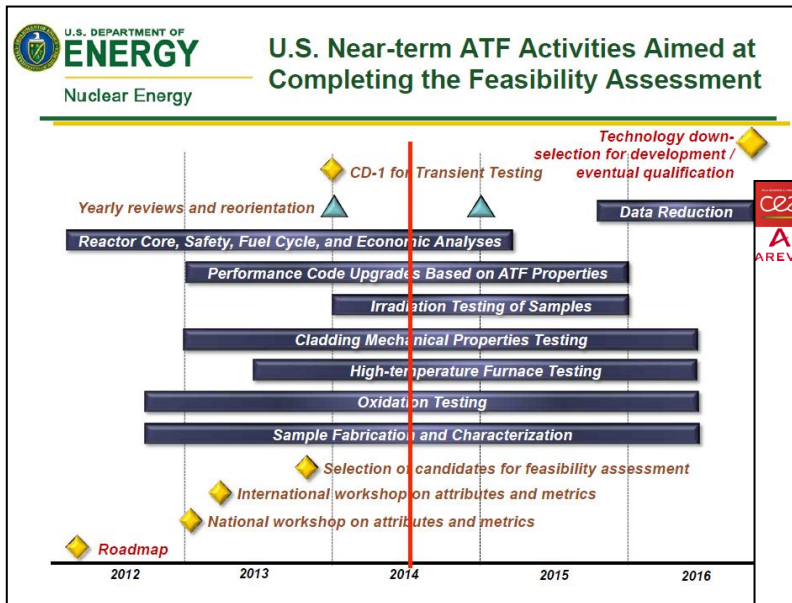
Uranium Silicide
Fabrication

Highest achieved
density:
94.6% theoretical
(11.5 g/cc)



Each concept has pros and cons across the spectrum of operating and transient conditions of interest. A systematic analytical and experimental evaluation is being performed during the feasibility studies.

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SYNERGIES WITH OTHER INTL PROGRAMS

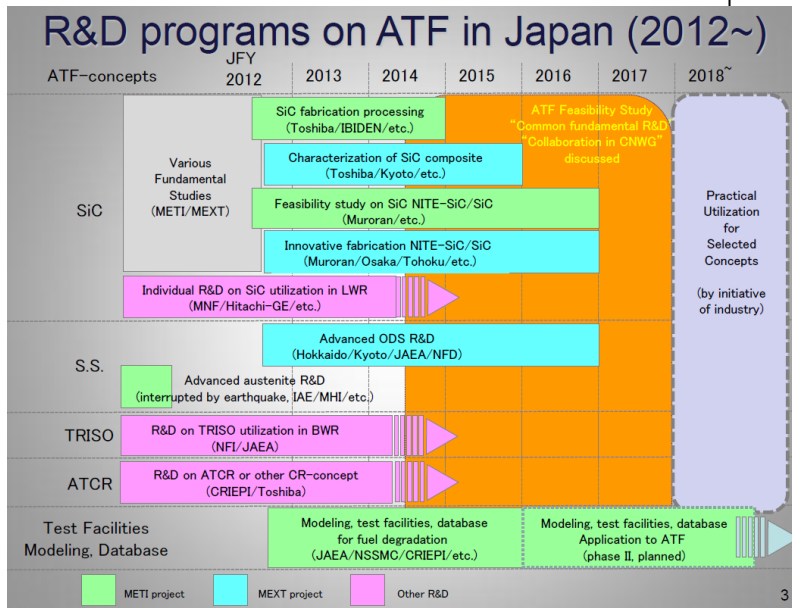
OECD-NEA Halden Reactor Project (WGFS)

- Proposal for irradiation testing of Cr-coated- Zy-4/UO₂ – fuel rodlets (White paper submitted)
- Goals
 - Step 1
 - Direct evaluation of under representative PWR conditions
 - Stability of the protective effect of coating
 - In environment
 - In environment + local deformation (special design for early contact)
 - Conditioning irradiated rodlet(s) for LOCA test in a second step
 - Step 2
 - LOCA test – Direct benefit evaluation
- Timing
 - 15 - 20 cm length clads may be short term delivered now
 - 50 - 60 cm clads could be fabricated end 2014 / early 2015
 - Discussion on detailed program and PIE expected around mid-2014

NUGENIA Technical Area 5 :

- several proposals to be coordinated around ATF cross evaluation goals and basic data acquisition (Falstaff, Forclad, Ramat, ...) → Call H2020 in 2014/9
- need of harmonization with other ATF initiatives

CEA, EDF, AREVA logos.



OECD-NEA Start-up meeting of the Expert Group on ATF of LWRs, April 28-29, 2014 11

EGATFL: 30 organisations from 13 member countries + PRC

Country	Organisation	Country	Organisation
Belgium	SCK-CEN	Rep. of Korea	KAERI
P.R. of China	CGN	Norway	OECD-Halden
Czech Rep.	ALVEL	Russian Federation	Kurchatov Institute
France	AREVA		NITI
	CEA	Switzerland	PSI
	EDF		Sweden
	IRSN	USA	
Germany	KIT		EPRI
Japan	CRIEPI		GE
	JAEA		INL
	Kyoto University		Univ. Illinois
	Muroran Inst. Techn.		Univ. Florida
	NFD		MIT
Toshiba	ORNL		
P.R. of China	CGN	UK	NNL
		International org.	IAEA

